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Tsiklauri et al.

[45] Date of Patent: * Jun. 11, 1996

[54] **METHOD AND APPARATUS FOR STEAM MIXING A NUCLEAR FUELED ELECTRICITY GENERATION SYSTEM**

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"Gas Turbines For Superheat In A Nuclear Power Plant", Proceedings of the American Power Conf., vol. 32, (1970), pp. 404-411.

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[*] Notice: The portion of the term of this patent subsequent to May 25, 2014, has been disclaimed.

[57] ABSTRACT

A method and apparatus for improving the efficiency and performance of a nuclear electrical generation system that comprises the addition of steam handling equipment to an existing plant that results in a surprising increase in plant performance. More particularly, a gas turbine electrical generation system with heat recovery boiler is installed along with a micro-jet high pressure and a low pressure mixer superheater. Depending upon plant characteristics, the existing moisture separator reheater (MSR) can be either augmented or done away with. The instant invention enables a reduction in T_{hot} without a derating of the reactor unit, and improves efficiency of the plant's electrical conversion cycle. Coupled with this advantage is a possible extension of the plant's fuel cycle length due to an increased electrical conversion efficiency. The reduction in T_{hot} further allows for a surprising extension of steam generator life. An additional advantage is the reduction in erosion/corrosion of secondary system components including turbine blades and diaphragms. The gas turbine generator used in the instant invention can also replace or augment existing peak or emergency power needs. Another benefit of the instant invention is the extension of plant life and the reduction of downtime due to refueling.

[21] Appl. No.: **392,659**

[22] Filed: **Feb. 23, 1995**

Related U.S. Application Data

[63] Continuation-in-part of Ser. No. 249,786, May 25, 1994, Pat. No. 5,457,721.

[51] Int. Cl.⁶ **G21D 1/00**

[52] U.S. Cl. **376/317; 376/402; 60/644.1; 60/653**

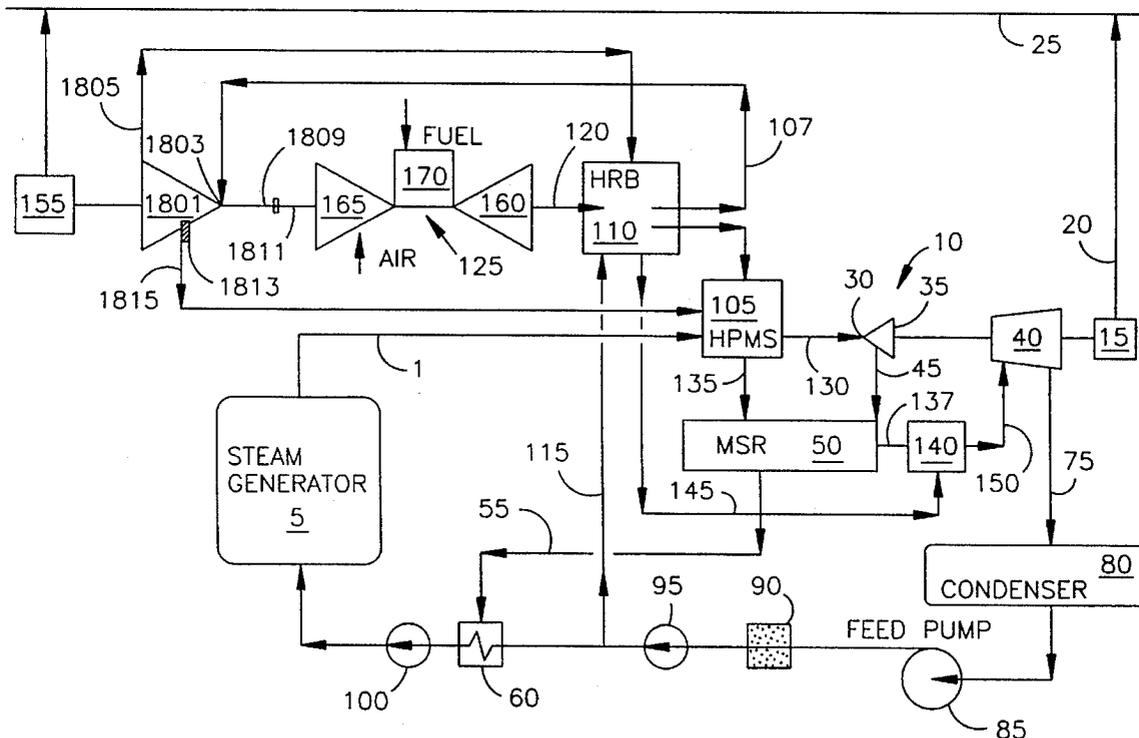
[58] Field of Search **376/317, 378, 376/391, 402; 60/644.1, 648, 653**

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24 Claims, 14 Drawing Sheets



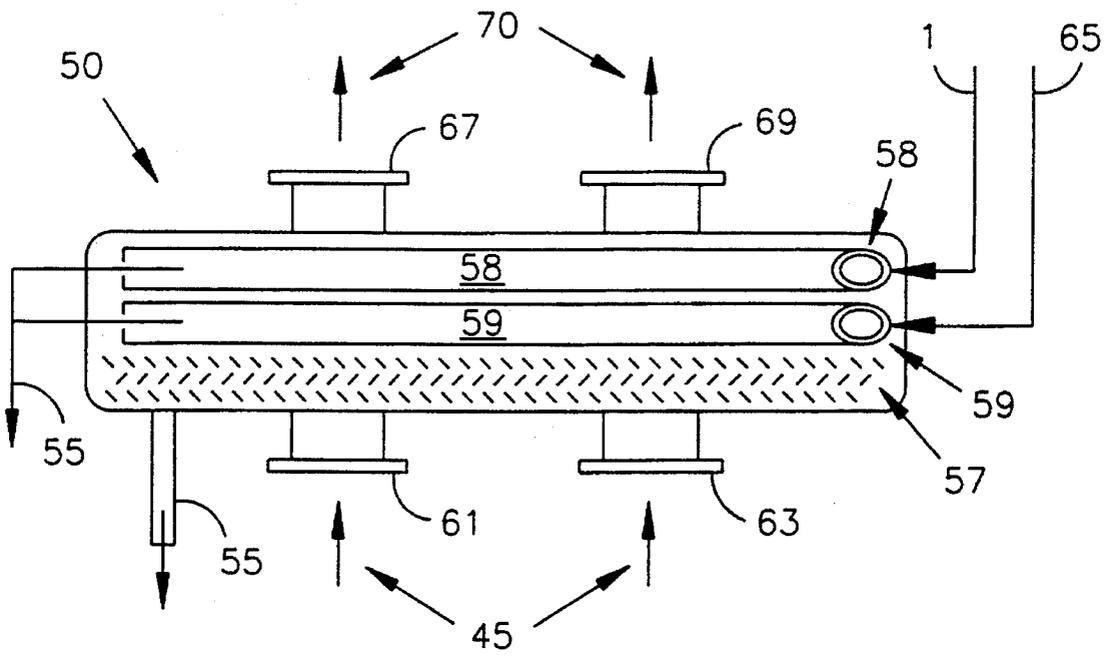


FIG 2
(PRIOR ART)

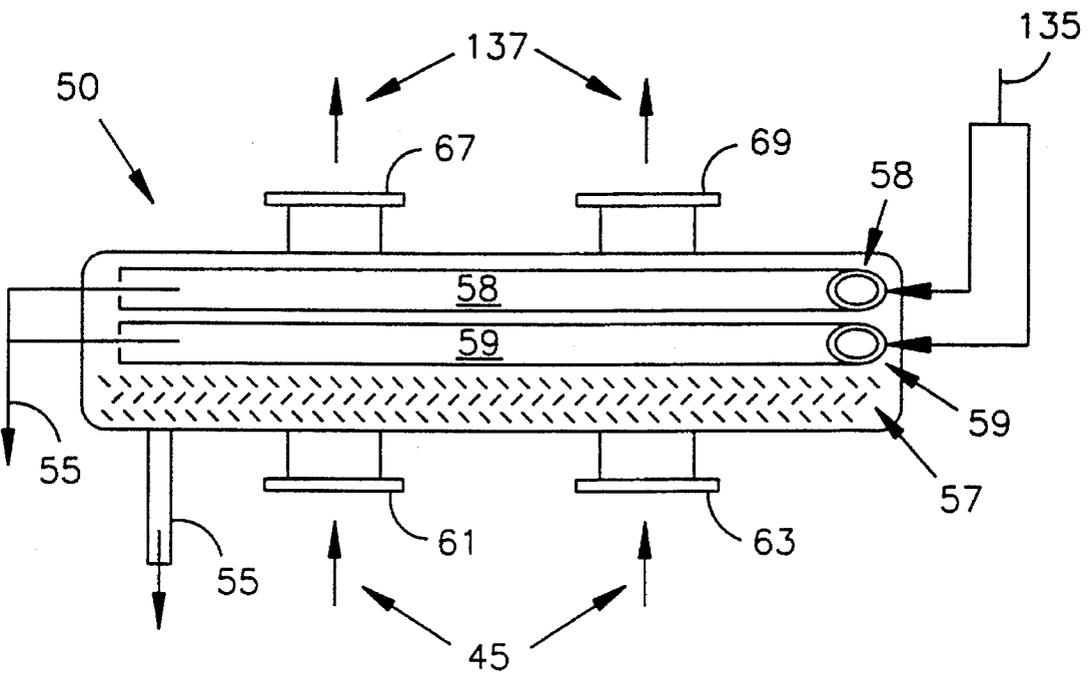
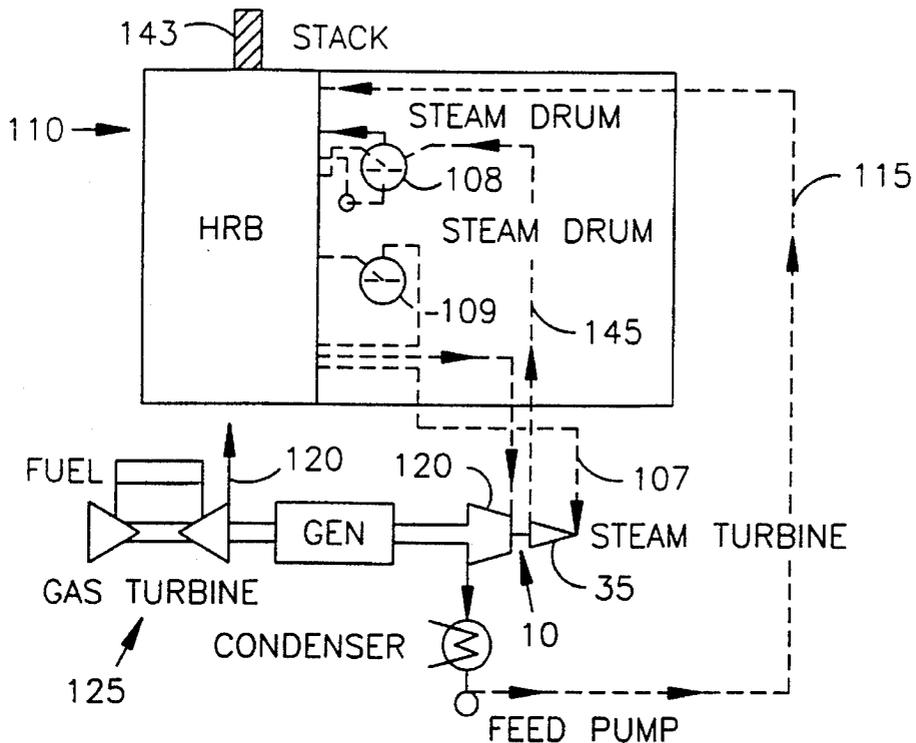


FIG 6



(PRIOR ART)
FIG 3

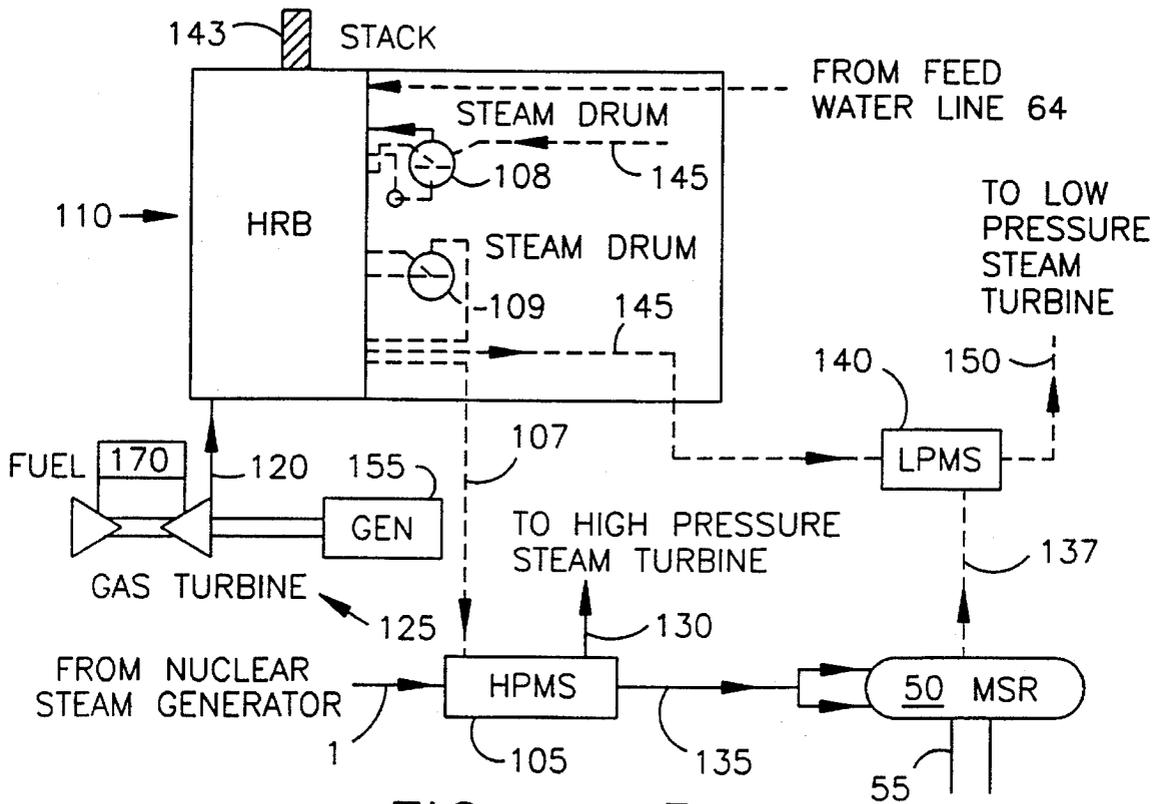


FIG 5

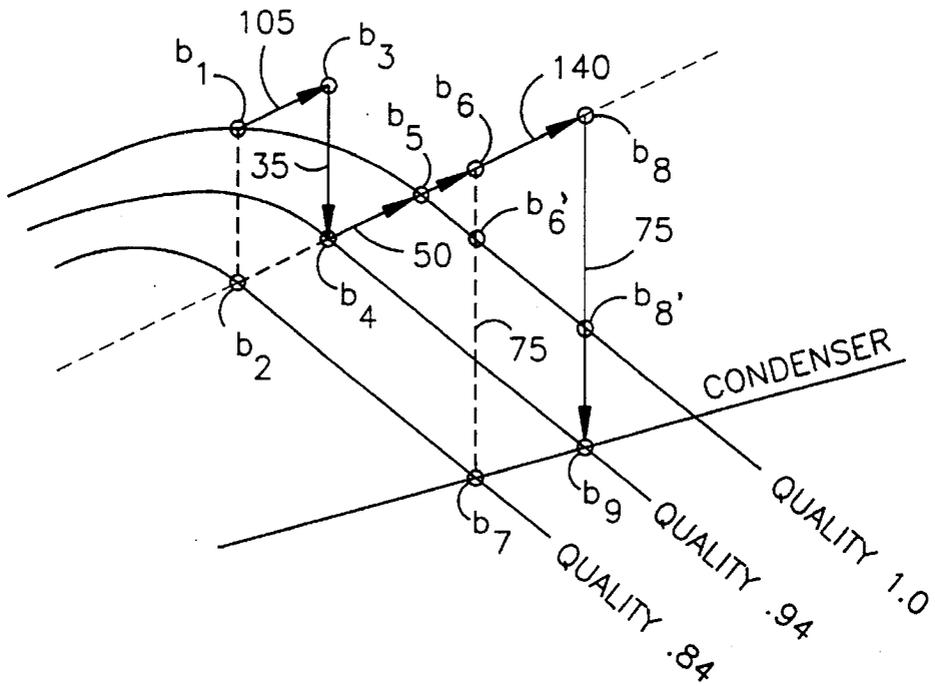


FIG 9b

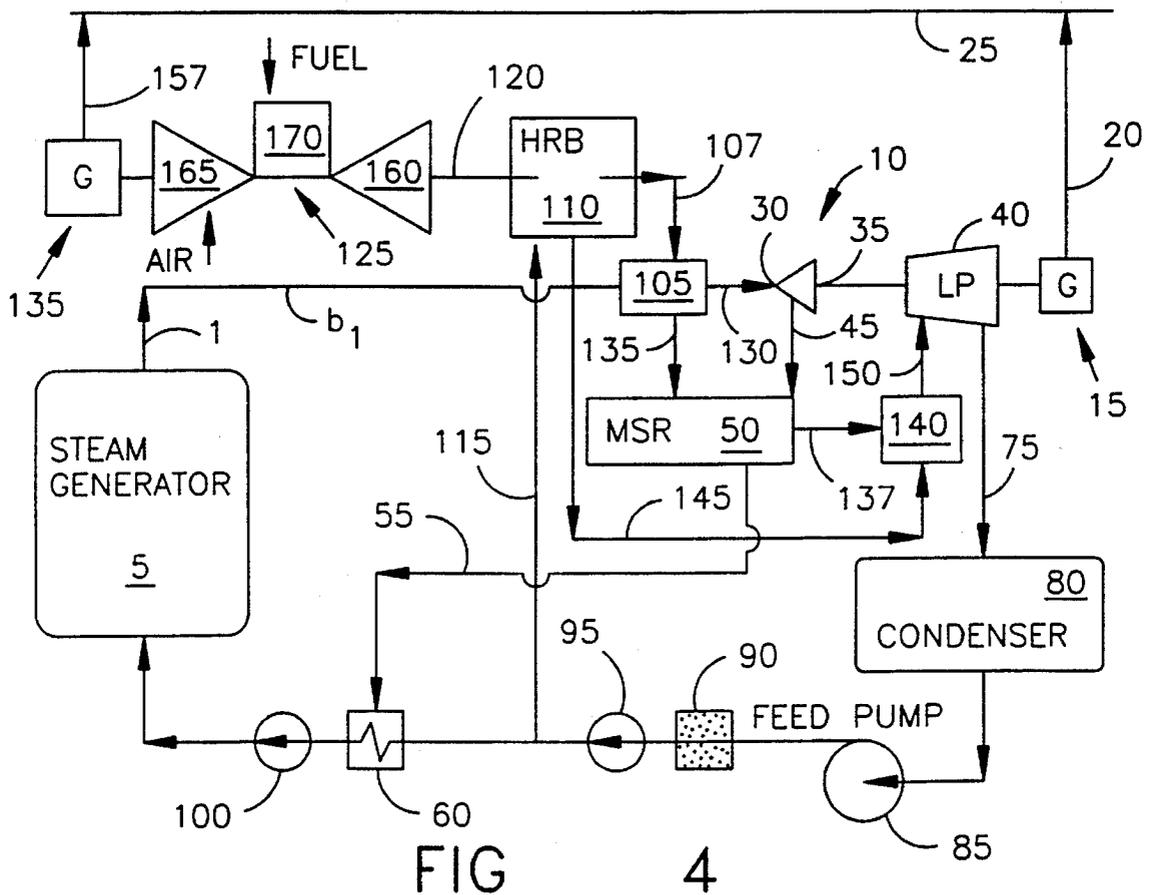


FIG 4

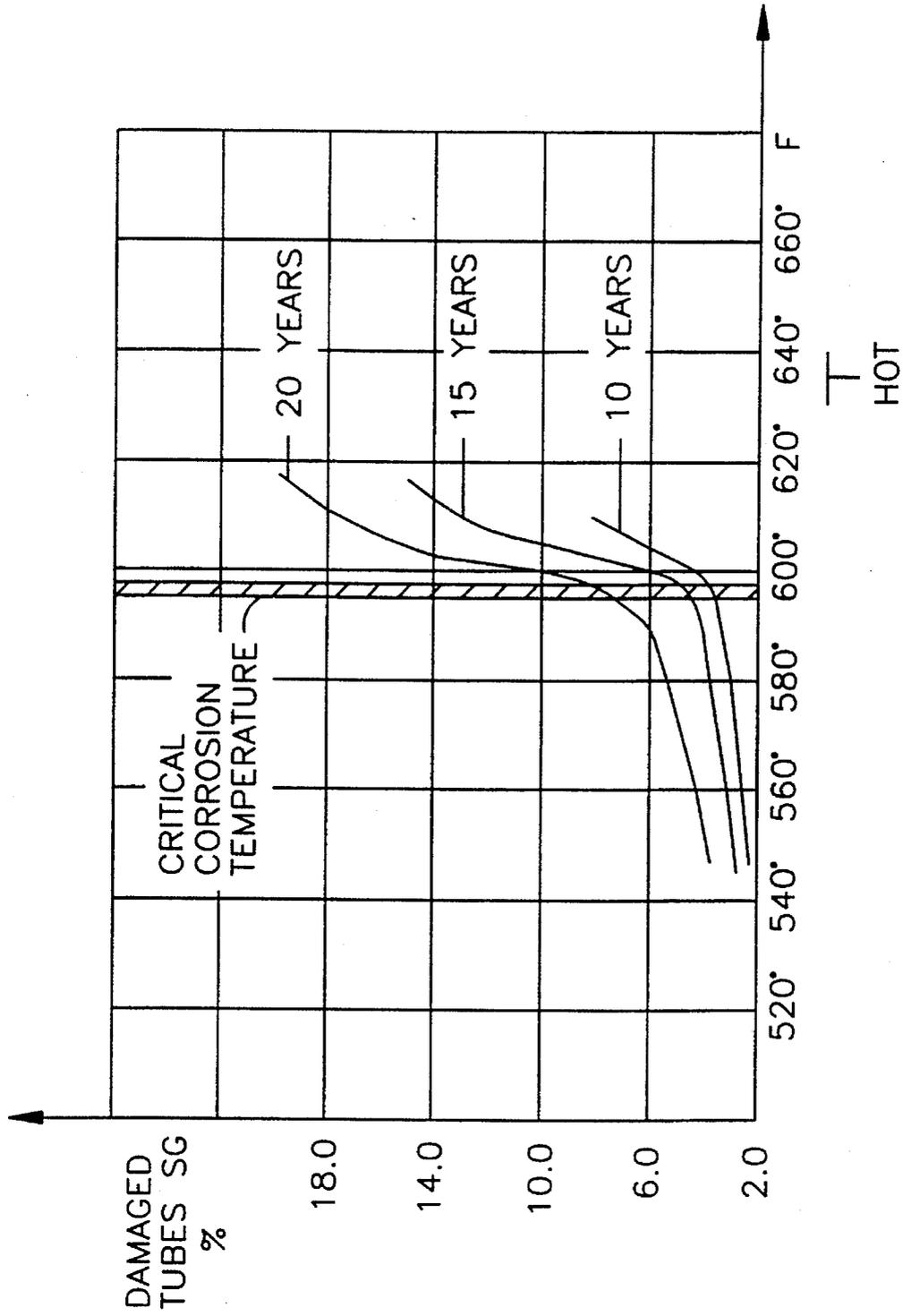
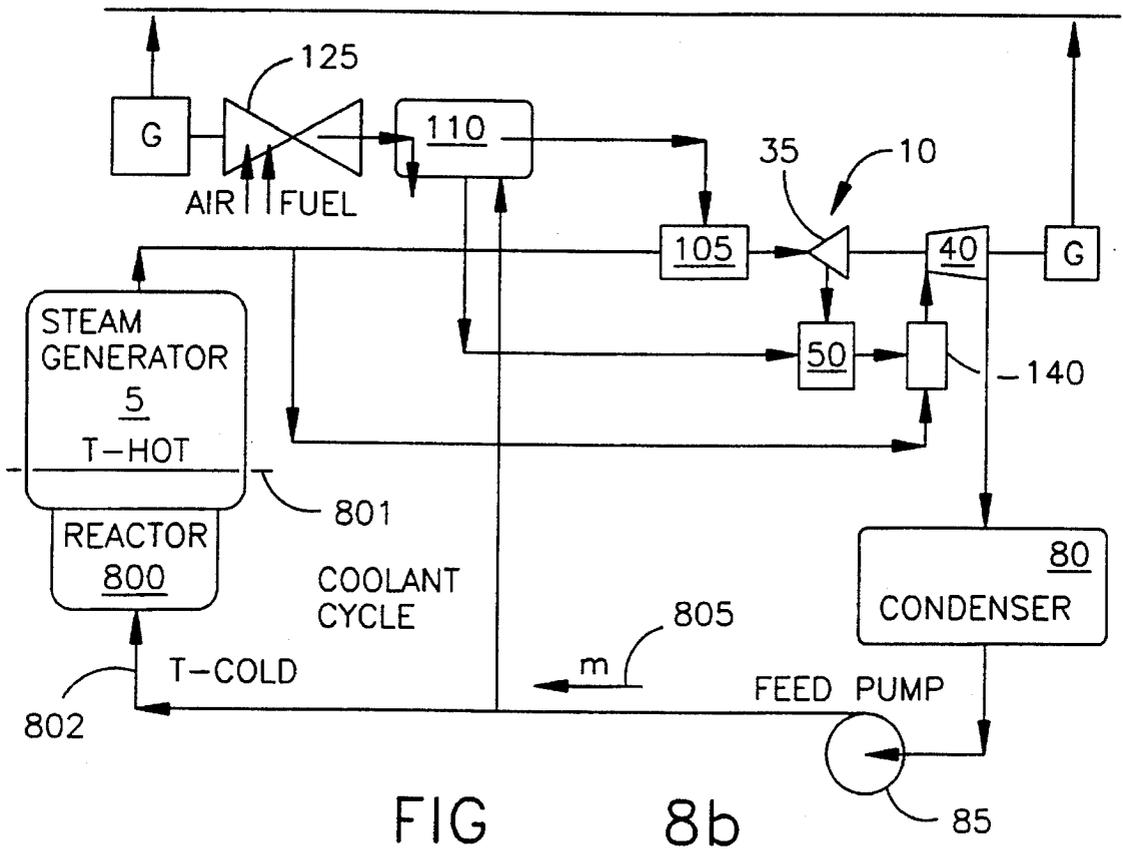
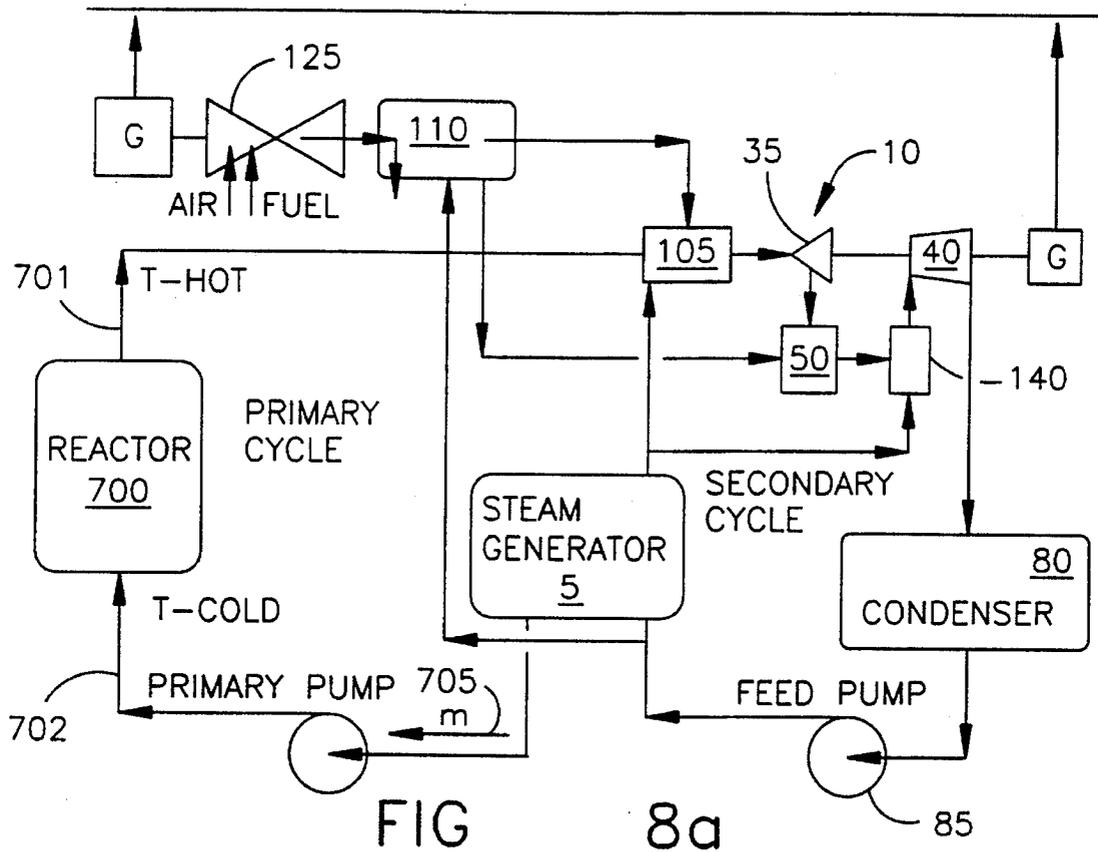


FIG 7



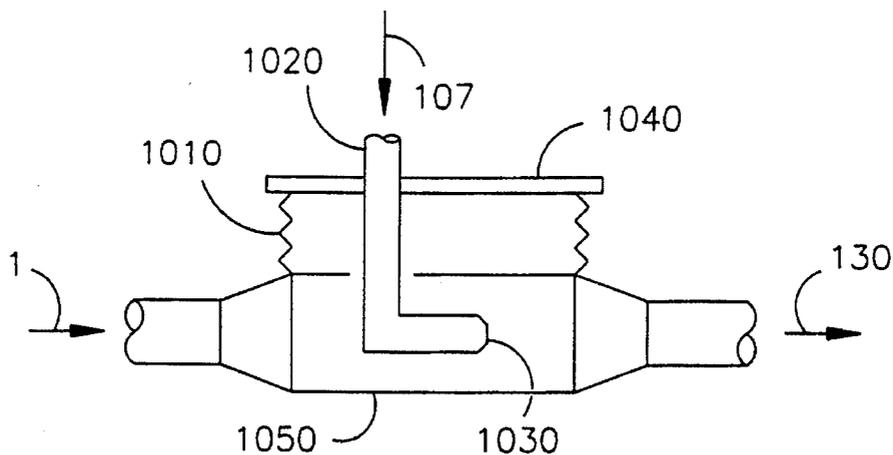


FIG 10 (PRIOR ART)

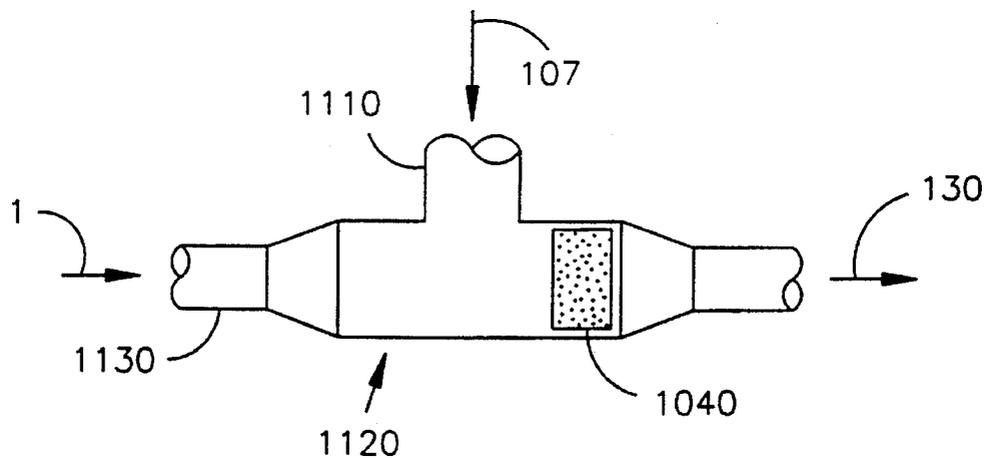


FIGURE 11 (PRIOR ART)

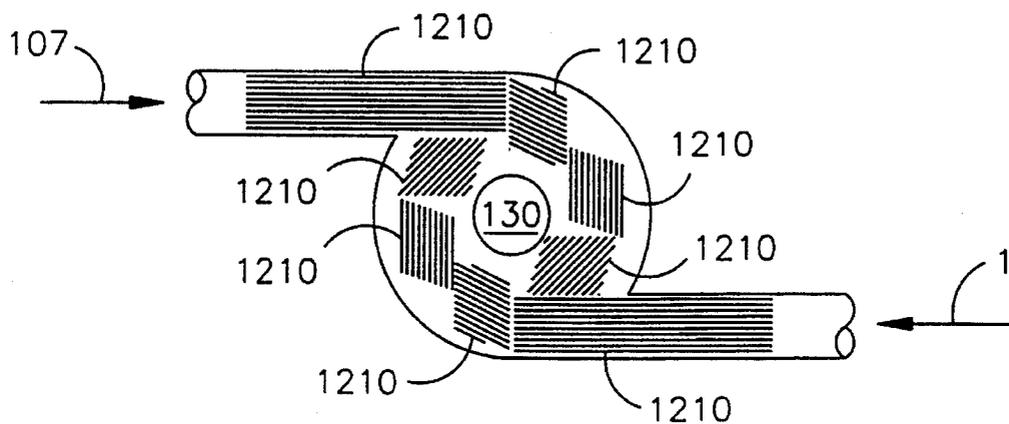


FIG 12 (PRIOR ART)

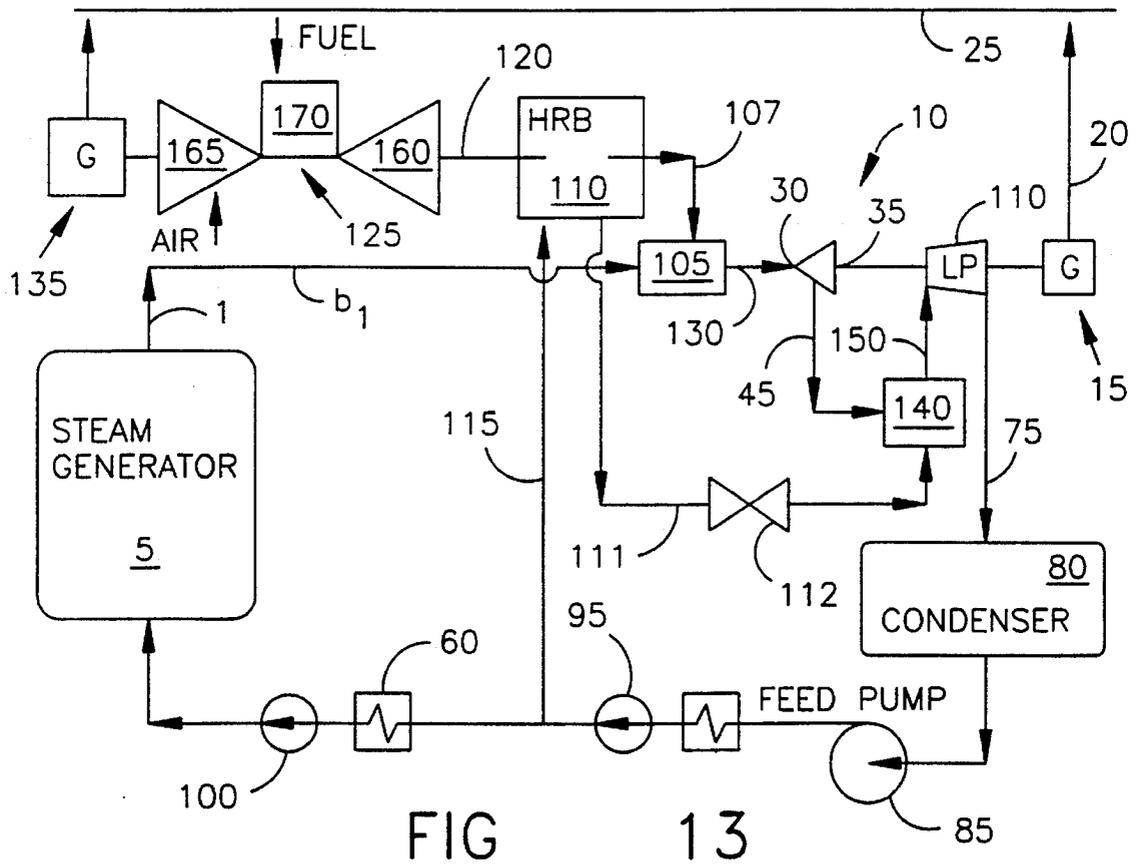


FIG 13

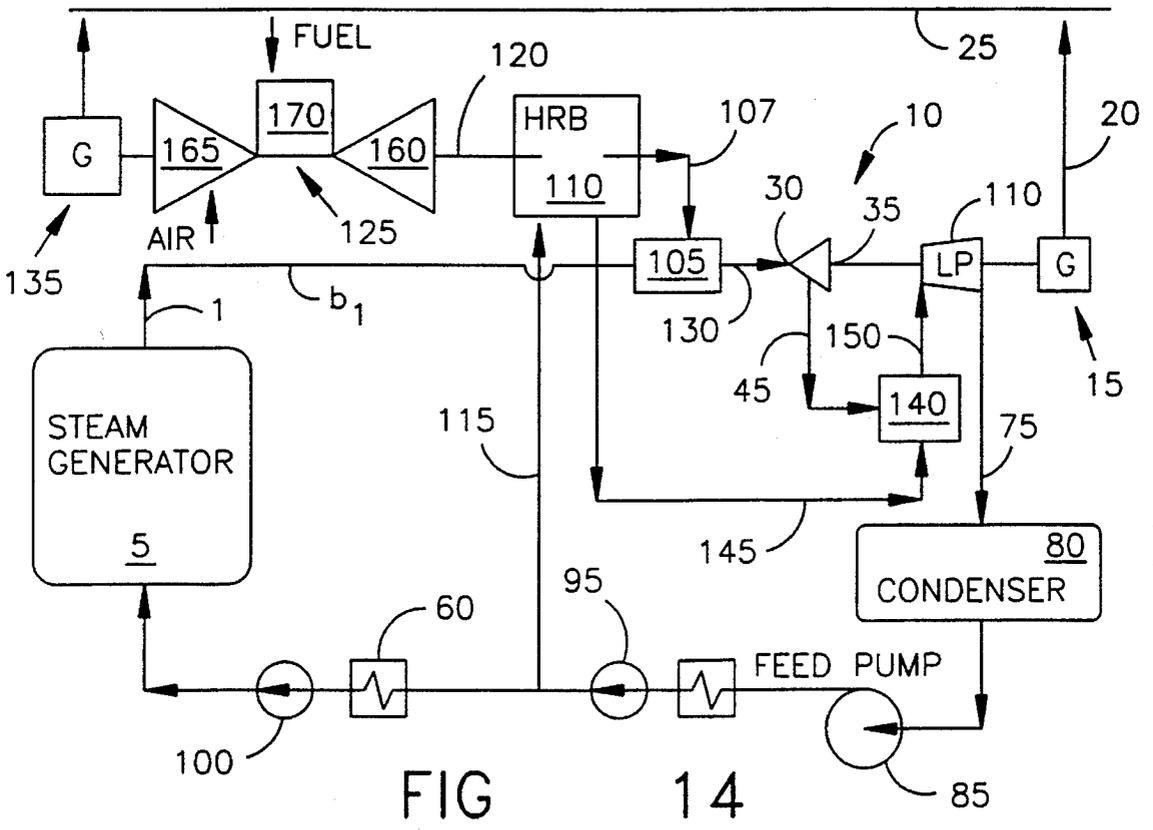


FIG 14

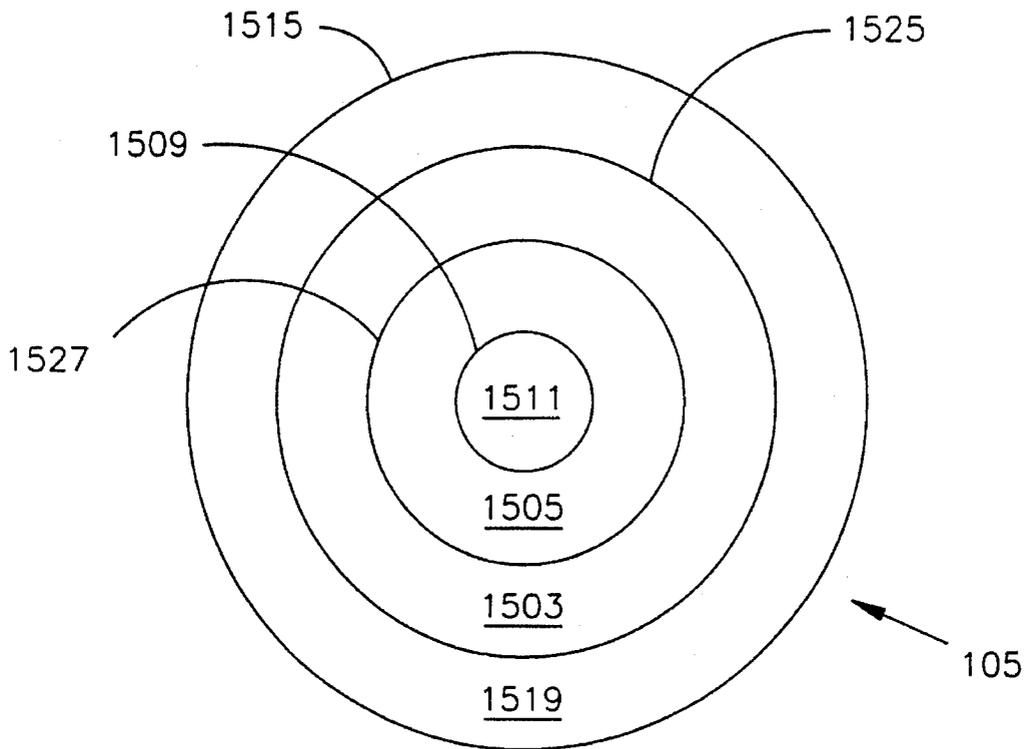


FIG 15b

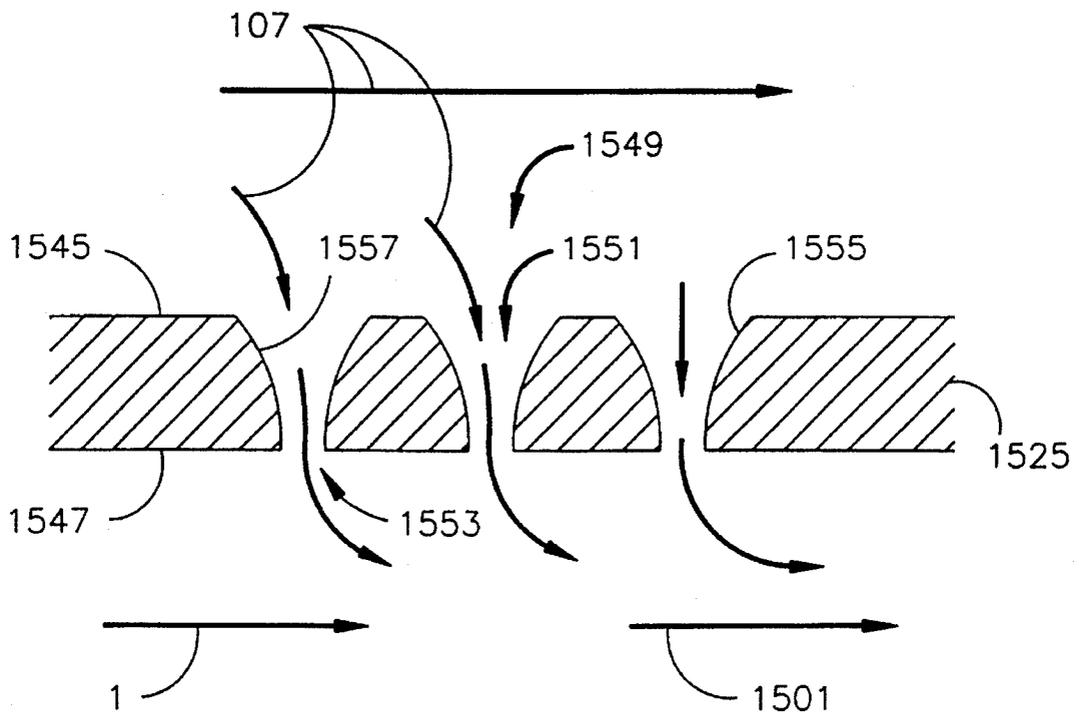


FIG 17b

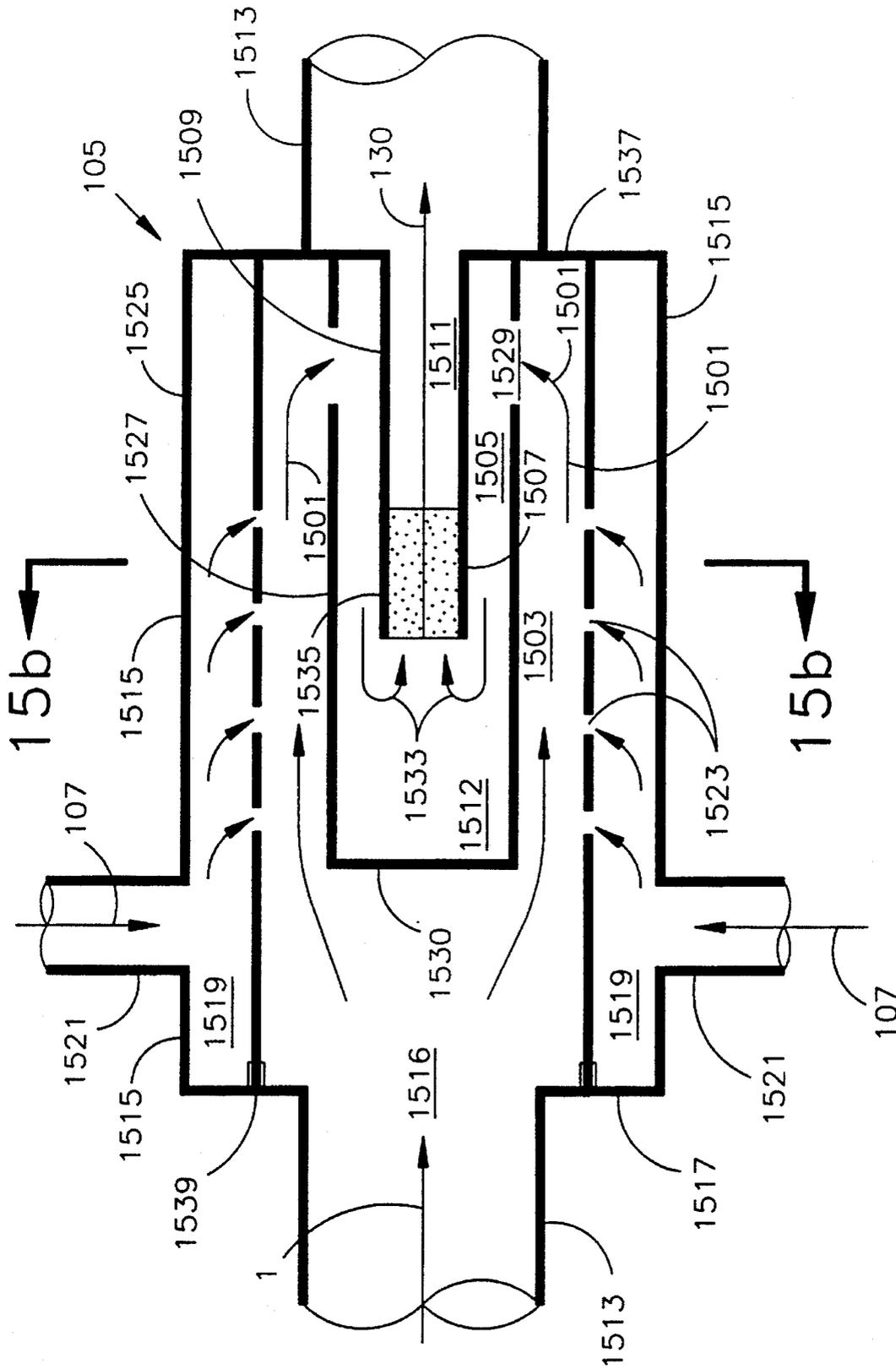


FIG 15a

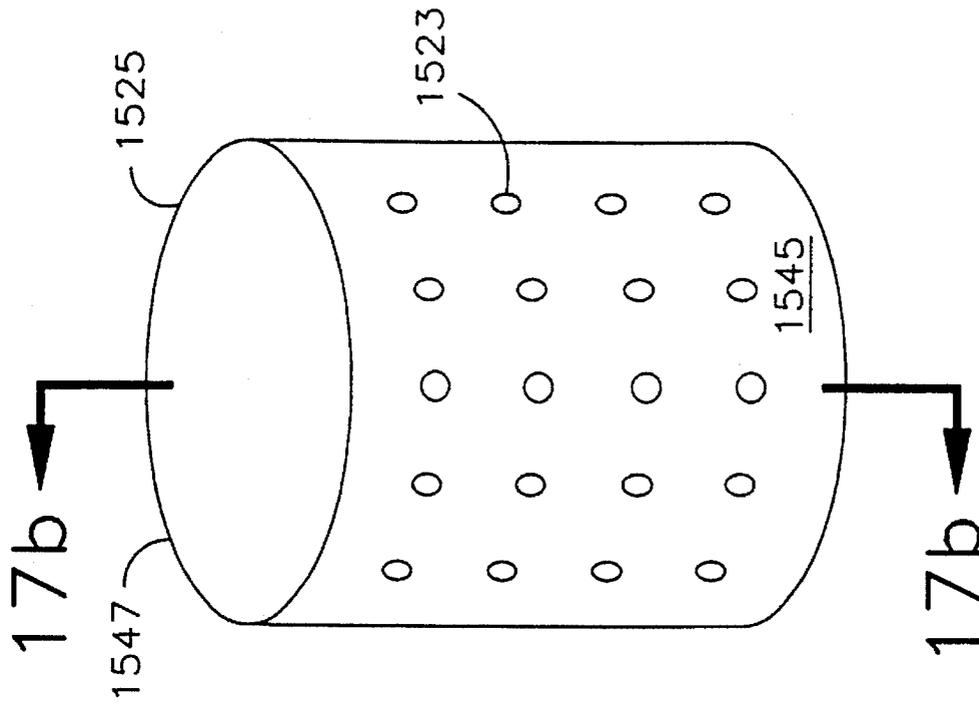


FIG 17a

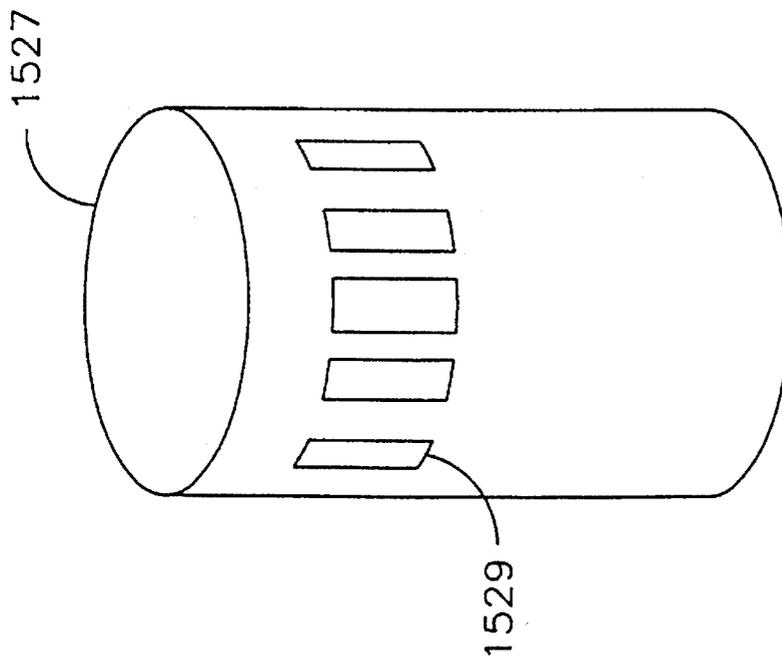


FIG 16

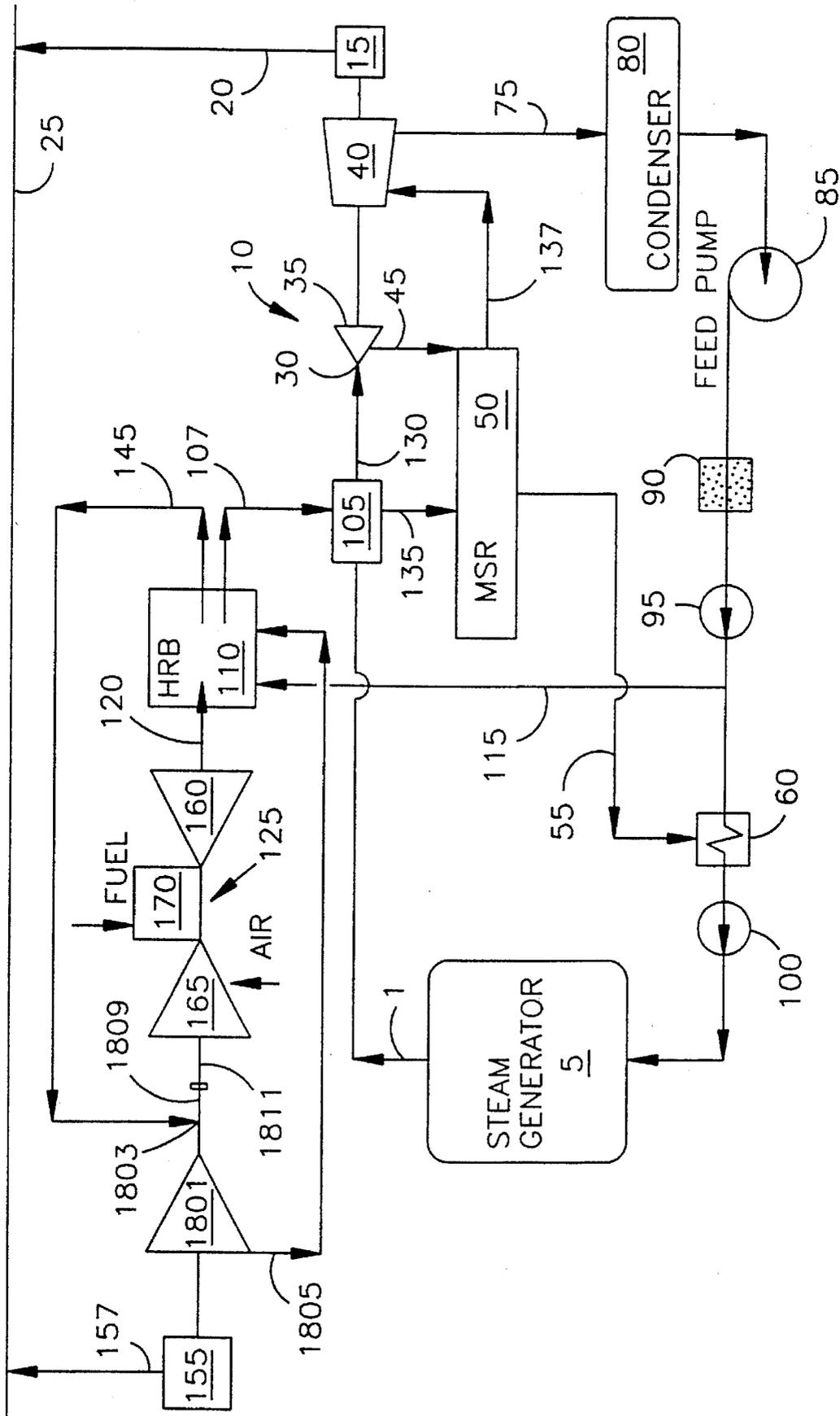


FIG 18a

METHOD AND APPARATUS FOR STEAM MIXING A NUCLEAR FUELED ELECTRICITY GENERATION SYSTEM

CROSS-REFERENCE TO RELATED APPLICATION

This application is a continuation-in-part of application Ser. No. 08/249,786, filed May 25, 1994, now U.S. Pat. No. 5,457,721.

FIELD OF THE INVENTION

The present invention relates generally to improving the efficiency, reliability, safety, steam mixing, and other operational characteristics of nuclear electrical power plants with a light water reactor. And more particularly the instant invention relates to mixing superheated steam from a heat recovery boiler of a gas turbine combined cycle steam system with main steam at the HP stage and the LP stage of the main turbine.

BACKGROUND OF THE INVENTION

In the operation of nuclear power plants, there are several important problems that significantly increase the cost of operations and maintenance. Some of these significant problems are: inherently low thermal efficiency, erosion-corrosion fuel storage and waste disposal limitations, equipment maintainability and reliability concerns as well as the limited fuel availability and the expensive requirements of storing spent fuel.

Various approaches have been tried to solve these problems separately, but few of the existing methods attack more than one of these concerns at a time.

Currently, in nuclear power plants (both pressurized water reactors or boiling water reactors), steam at the inlet of the turbine is saturated. This results in lower turbine efficiency and intensive erosion-corrosion problems.

There have been several attempts to use single or multiple fossil reheaters to treat the steam of a operating nuclear power plant. However, while higher steam temperatures due to fossil heating between the low pressure turbine stage and the high pressure turbine stage have been achieved, higher efficiency still has not been demonstrated. One such attempt is U.S. Pat. No. 5,361,377 to John A. Miller. The Miller heat exchanger method and apparatus of treating the steam claims an increase in efficiency by use of a fossil fired reheater. A problem with this method is that the fossil reheater still consumes fuel in a process that is inefficient in itself. In other words a big part of the heat from the burning of the fossil fuel goes up the stack. A process is needed that produces electricity, and utilizes the waste heat from that process to treat the steam in an efficient manner.

Mixing of the steam going from the LP stage to HP stage of the main turbine with high enthalpy steam is a solution that brings all of these advantages.

It is generally recognized that for mixing to be done in non-isothermal fluids, in boundary flow conditions, a pipe length of about 50 diameters is required. In a typical nuclear power plant the typical main steam pipe outside diameter is approximately 0.8 meter. This would require a pipe of 40 to 50 meters in length to achieve through mixing. Therefore, a more efficient means of mixing is needed to mix the steam from a heat recovery boiler with main steam.

It is therefore an object of the instant invention to treat the steam prior to entering the HP stage of the main turbine, then again before entering the LP stage by mixing the nuclear steam with high enthalpy steam from an efficient process.

SUMMARY OF THE INVENTION

It is therefore an object of this invention to provide a means to improve the efficiency, availability, safety, and steam mixing, and economy of nuclear power generation systems. And more particularly an improved means for providing superheated steam to the steam turbine driving the electrical generator.

It is a more particular object of the invention to allow for lower hot leg temperatures (T_{hot}) in the primary cycle of an existing nuclear power plant. The lower T_{hot} would reduce the rate of erosion-corrosion and extend the lifetime of PWR steam generators.

It is a further object of the invention to extend plant life by enabling the delivery of the same amount of electricity for a smaller amount of heat generated by nuclear reaction. In other words, the invention is a combined nuclear-gas turbine cycle that can be used to, inter alia, reduce corrosion damage to the nuclear steam generator by operating the outlet reactor hot leg temperature below the critical corrosion temperature. Corrosion is significantly reduced by operating steam generators at less than 600° F. (315° C.). With the instant invention nuclear power plants can operate below 600° F. (315° C.) without derating the electrical output of the station. The advantage of reduced corrosion is due to reduced steam generator temperature caused by an increased thermal efficiency of the resulting steam cycle.

One factor that causes an increase in efficiency is that the additional electrical output from the gas turbines further adds to the electrical grid output of the station. In the instance where the hot leg temperature is reduced below the critical steam generator corrosion temperature, the thermal power of the reactor will be lower, resulting in a potentially longer operating cycle time between reactor refueling cycles.

This enables a longer period between refueling which is getting progressively more expensive. The disposal of spent nuclear fuel and waste, which is now a major concern of nuclear industry, will be reduced significantly.

In addition, lower reactor power avails opportunities to perform maintenance on many key apparatuses in the secondary system (the steam cycle in PWRs and the coolant cycle in BWRs) while the reactor is at power. By reducing corrosion and erosion the down time is decreased thus enabling the plant to operate with less down time. It is well known in the art that the fuel replacement cost during downtime is from several hundreds of thousands of dollars to one million dollars for each day, depending upon the plant.

When the reactor is operating at a lower thermal power, there are, in essence, built-in spare components with the arrangement of currently installed equipment.

To underscore the importance of erosion and corrosion in steam equipment it should be noted that maintenance of many secondary-side (i.e., steam handling) components are often the source of unplanned outages.

In the instant invention, the steam turbine has higher efficiency and reduced erosion-corrosion. The reactor steam, prior to entering the high pressure stage of the main turbine, is superheated by a high-pressure-mixer-superheater

(HPMS) that utilizes the highly superheated steam from the heat recovery boiler (HRB) of the gas turbine combined cycle. The steam is then conditioned, according to need, by several different components.

In the most commonly used application to improve the efficiency of an operating nuclear power plant the steam would next go to the moisture separator reheater (MSR), that is a common component of existing plants. The MSR can be bypassed depending upon the existing steam quality from the high pressure (HP) stage of the main turbine. The steam is next improved for use by the low pressure (LP) stage of the main turbine. When required this is done in a low pressure mixer superheater (LPMS) which mixes steam from the MSR (when used) with HRB steam. The instant invention has a marked increase in the portion of the LP stage of the main turbine that is driven by superheated steam. This enables a reduction in corrosion and erosion.

The invention also provides an additional avenue to make nuclear stations safer due to the addition of an operating source of alternative emergency power (i.e., the gas turbines). Additionally, the size of the gas turbine and generator is not limited and the instant invention therefore readily lends itself to operation in conjunction with a large gas turbine and generator that could function as an independent source of power during peak times or alone to supply a grid.

BRIEF DESCRIPTION OF THE DRAWINGS

FIG. 1: Is a schematic representation of the steam cycle of a typical nuclear reactor electric generation plant.

FIG. 2: Is a sectional view of a diagrammatic representation of the flow path in a MSR used in a typical nuclear electric generation plant.

FIG. 3: Is a schematic representation of an HRB when used in a combined cycle power plant.

FIG. 4: Is a schematic representation of a typical nuclear reactor electric generation plant utilizing the invention to improve performance.

FIG. 5: Is schematic representation of the flow path in a HRB used in a nuclear electric generation plant utilizing the instant invention.

FIG. 6: Is sectional view of a schematic representation of the flow path in a MSR used in a nuclear electric generation plant utilizing the instant invention.

FIG. 7: Is a graph of the percentage of tubes plugged due to corrosion versus time in operation with various temperature curves being illustrated.

FIG. 8a: Is a schematic representation of the primary and second cycles of a pressurized water reactor electrical generation plant utilizing the instant invention.

FIG. 8b: Is a schematic representation of the primary and second cycles of a boiling water reactor electrical generation plant utilizing the instant invention.

FIG. 9a: Is a portion of a Mollier diagram that represents the operation of a typical nuclear reactor electric generation plant.

FIG. 9b: Is a portion of a Mollier diagram that represents the operation of a typical nuclear reactor electric generation plant utilizing the invention to improve performance.

FIG. 10: Is a schematic sectional view of a tube steam mixer superheater jet chamber with a bellows arrangement.

FIG. 11: Is a schematic sectional view of a piping tee with a honeycomb mixer section inserted.

FIG. 12: Is a schematic sectional view of a vane-vortex steam mixer superheater arrangement.

FIG. 13: Is a schematic representation of a typical nuclear reactor electric generation plant that has a heat recovery boiler that has only pressure steam output.

FIG. 14: Is a schematic representation of a typical nuclear reactor electric generation plant that has a heat recovery boiler that has only two steam pressure outputs and no moisture separator reheater.

FIG. 15a: Is a schematic cross sectional representation of the micro-jet mixer.

FIG. 15b: Is a longitudinal view of the micro-jet mixer through lines 16—16.

FIG. 15c: Is a longitudinal view of the micro-jet mixer through lines 16—16 that illustrates dimensions and volumes.

FIG. 16: Is an isometric view of the slotted baffle illustrating the baffle slots.

FIG. 17a: Is an isometric view of the perforated baffle illustrating the micro-jets.

FIG. 17b: Is a longitudinal view of the perforated baffle wall illustrating the nozzle inlets and outlets.

FIG. 18a: Is a schematic representation of an embodiment of the instant invention installed in a typical nuclear reactor electric generation plant that employs a booster turbine and does not use a LPMS.

FIG. 18b: Is a schematic representation of an embodiment of the instant invention installed in a typical nuclear reactor electric generation plant that employs a booster turbine and a LPMS.

DESCRIPTION OF THE PREFERRED EMBODIMENTS

Nuclear power electrical generation plants have a tremendous potential to produce energy. However, these plants have technical problems that prevent them from reaching their full energy production and from lasting to the end of their design lifetime. These include corrosion due to the high temperatures that they must operate at, and erosion of turbine and secondary system components to name a few. The instant invention enables the lowering of primary T_{hot} temperature, steam generator temperature, and the increasing of steam quality that greatly reduces erosion of piping and components as well as turbine blades and diaphragms.

EXISTING STEAM PERFORMANCE

Referring to FIG. 1, the typical nuclear power electric generation system that utilizes a light water reactor will typically operate with wet main steam 1. Wet main steam 1 for this type of system, including both pressurized water reactors and boiling water reactors, is typically supplied by a steam generator 5 to drive the main turbine 10 that is the mechanical force to operate an electrical generator 15. Electrical power 20 generated by these systems is supplied to a grid 25 that purchases the electrical power 20.

Wet main steam 1 arriving at the main turbine 10 produces moisture problems such as corrosion and erosion of the turbine blades and diaphragms that limit plant life and are extremely costly in down time, replacement, and operational limitations. Main steam 1 at the entrance 30 to the main turbine 10 will typically be from 0.2% in moisture and be from 970 to 1030 psig (68 bar to 72 bar). The main turbine 10 typically consists of two stages, the high pressure (HP) 35 stage and the low stage (LP) Stage 40. After work is extracted from main steam 1 in the HP stage 35 the char-

acteristics of the HP exhaust steam **45** must be enhanced to improve utilization of steam by the LP stage **40**.

Currently, nearly all the nuclear power plant turbines, operating on wet main steam **1**, whose expansion starts from the saturation curve (quality 1.0), are equipped with moisture separator reheaters (MSR) **50**. Without MSRs **50** the HP exhaust steam **45** leaving the HP stage **35** may contain up to 25% of water (quality less 75%) that is too damaging for operation because of the erosion and corrosion. The MSR **50** not only improves the turbine **10** reliability (reduce the erosion of blades, housings, and diaphragms), but also improves efficiency of low pressure (LP) stage **40** of steam turbine **10** as much as 2 to 4%.

The construction of the MSR **50** is not critical to the instant invention and is well known in the art. FIG. 2 illustrates the typical construction of a MSR **50**. Now referring to FIG. 2, moisture **55** that is typically removed by chevron separating elements **57** from the HP exhaust steam **45** is returned to the feed system through a reheater **60**. After the chevron separating elements **57** the HP exhaust steam **45**, it is heated by at least one set of tube bundles **58,59**. Most MSRs **50** in service today have two sets of tube bundles **58** and **59**, and HP exhaust steam **45** enters on the bottom in dual inlet portals **61** and **63**, and exits the top of the MSR **50** through dual exit portals **67** and **69**.

Once the moisture **55** is separated from the HP exhaust steam **45** it is first heated by using extraction steam **65**, then it is heated by main steam **1** with a set of tube bundles **59**, and is then heated by tube bundle **58** that is supplied with main steam **1**.

Extraction steam **65** is then directed from the MSR **50** as MSR exhaust steam **70** and is then sent to the LP stage **40** of the main turbine **10**, and is slightly superheated, and is typically from 85 to 185 psig (6 to 13 bar). The MSRs **50**, one of the most unreliable components in the entire generation system, are extremely expensive, and are usually one-third the cost of the main turbine **10** itself.

Also shown in FIG. 1 are other components that are typically found in light water nuclear reactor power generation systems. For example, turbine exhaust will proceed through the plenum **75** that connects the main turbine **10** and the condenser **80**, to feed pump **85**, to demineralizer **90**, and then finally to pumping stages, **95** and **100**, before being returned to the steam generator **5**. Nuclear-electric generating systems of this type typically have an efficiency of about 33 to 35% with a heat rate of about 10,000 Btu/Kwh. Existing combined-cycle-electric plants currently utilize a heat recovery boiler **110** (HRB) to increase the efficiency. FIG. 3 illustrates how an HRB **110** would be utilized in a typical combined-cycle-electric plant.

ENHANCEMENT OF OPERATIONAL PERFORMANCE

FIG. 4 illustrates the overall plant schematic for a nuclear plant with the instant invention installed in the downstream portion of main steam **1** piping **1513** of the steam generator **5**. It will become apparent how the instant invention improves the operational characteristics of HP inlet mixed steam **130** entering the entrance **30** to the HP Stage **35** and the LP stage **40** of the main turbine **10**. It should be noted in both FIGS. 1 and 4, main steam **1** denotes the main steam piping **1513** and the flow of main steam **1** within that piping.

Main steam **1** from the steam generator **5** arrives at a high-pressure-mixer superheater (HPMS) **105** where it is superheated to the temperature from 570° to 680° F.

(299°–360° C.) by the high enthalpy-high pressure steam HRB outlet steam **107** from a heat recovery boiler (HRB) **110**.

USE OF HRB IN CONVENTIONAL APPLICATIONS

Use of a HRB **110** in a conventional combined-cycle plant, is illustrated in FIG. 3 and is well known in the art. The cycle starts with water **115** that is returned to HRB **110** from the condenser **80** by the discharge of the feed pump **85** where it is heated by the exhaust **120** from a gas turbine **125**. The fuel utilized by the gas turbine is not germane to the invention.

The exhaust gas **120** from the gas turbine **125** arrives to heat recovery boiler (HRB) **110**, where two pressures of steam are generated and then superheated. High pressure and high enthalpy, HRB high pressure steam **107** is generated in high pressure loop **109**; the HRB low pressure steam **145** is generated in low pressure loop **108**. The construction of HRB **110** is not critical to the present invention and is well known in the art.

The gas turbine consists typically of several components: gas turbines **160**, compressors **165**, and burners **170**. Operation of gas turbines is well known in the art. The gas turbine **125** is used to drive a generator **155** that supplies electricity **157** to the grid **25**. This increases the overall plant efficiency to about 42 to 45% when the heat range is from 8200 to 7600 Btu/Kwh.

USE OF HRB IN THE INSTANT INVENTION

Use of a conventional HRB **110** as described above is contemplated by the instant invention. However, the steam generated in the HRB will be used in a different manner.

The HRB high pressure steam **107** exits with high pressure (typically from 1000 to 1315 psig (93 bar)) and high temperature (typically from 830° to 935° F. (444°–500° C.)), and is sent to the high pressure mixer superheater (HPMS) **105** to mix and to superheat main steam **1**.

The HRB low pressure steam **145** exits with a pressure (typically from 80 to 200 psig (14 bar)) and high temperature (typically from 830° to 935° F. (450°–500° C.)), and is sent to the low pressure mixer superheater (LPMS) **140** to mix and superheat MSR discharge steam **137**.

Referring to FIGS. 18a and 18b, when it is desirable not to use a LPMS **140** the HRB high pressure steam **107** pressure should be boosted to approximately 2000 psig (140 bar), and the temperature should be from 830° to 1050° F. (444°–565° C.).

USE OF MSR IN CONJUNCTION WITH INSTANT INVENTION

Now referring to FIG. 6, the MSR **50** that is already installed in the plant can be used. The construction of the MSR **50** is not critical to the instant invention and is well known in the art. Moisture **55** that is removed from the HP exhaust steam **45** is returned to the feed system through a reheater **60**.

Once the moisture **55** is separated from the HP exhaust steam **45** it is reheated by using HPMS outlet steam **135** from the HPMS **105**, becoming MSR discharge steam **137**. The MSR discharge steam **137** is then sent to the LPMS **140** where it is superheated by the high enthalpy HRB low pressure steam **145** from the HRB **110** yielding high enthalpy low pressure steam, referred to as LPMS. This

discharge steam **150** is then sent directly to the LP stage **40** of the main turbine **10**. This treatment of HP exhaust steam **45** before entering the low pressure stage **40** of the steam turbine **10** results in high quality steam in flow passes, reducing erosion of rotor blades and diaphragms and increasing in efficiency by 3 to 5 percent.

GAS TURBINE AS A GENERATOR

Now referring again to FIG. 4, the gas turbine **125** is well known in the art as a reliable source of electrical energy. This enables the same turbine **125** that is used to supply heated exhaust **120** to act as the source of electrical power **157** for use in emergency situations. This emergency electrical power **157** would be in addition to, or to supplement, the diesel generators that are typically installed in nuclear power stations.

The gas turbine generator heat is the heat available in the gas turbine exhaust **120**. The amount of heat is only critical to the invention as long as it provides sufficient energy to make sure the heat recovery boiler high pressure steam **107** has sufficient enthalpy to increase the operation characteristics of the steam at the turbine inlet **30** and to supply the needed energy to the LPMS **140**. Any energy available above this amount can be sent up the stack **143** as shown in FIG. 5. Therefore, the gas turbine generator can be used as an independent source of electrical power **157** to the grid **25**. This enables the dual use of a gas turbine electrical generator set **155**. One use is to supply energy for the instant invention, and another use is as an independent electrical energy source to supply electricity to the grid **25**.

GRAPHICAL REPRESENTATION OF TYPICAL PLANT PERFORMANCE

The Mollier chart for the typical steam cycle of a typical light water reactor electric power generation system is illustrated in FIG. 9a. The diagram in FIG. 9a corresponds to equipment that is illustrated in FIG. 1.

Expansion in the HP stage of the Main Turbine (a_1 - a_2)

Main steam **1** expands in the HP stage **35** of the main turbine **10** until it exits as HP exhaust steam **45**. This line on FIG. 9a represents a decrease in quality from about 1 to about 0.85, and is illustrated as going from point a_1 to point a_2 .

Removal of Moisture from HP exiting wet steam in the MSR (a_2 - a_4)

In the MSR **50** the moisture **55** from HP exhaust steam **45** is first removed by chevron separating elements **57** that increase the quality to about 0.98 that is graphically represented in FIG. 9a by the line from point a_2 to a_5 .

Removal of Moisture from HP exiting wet steam in the MSR (a_4 - a_6)

After the moisture **55** is separated, the exhaust steam **45** is heated in two stages. The first is by extraction steam **65** and then by main steam **1**. This removes all remaining moisture or superheats the HP exhaust steam **45** to point a_5 or a_6 depending upon the operational characteristics in that plant at that time. At this point the HP exhaust steam **45** is typically at a temperature of about 570° F. (299° C.), and the MSR **50** exhaust steam **70** is typically at a temperature of about 500° F. (260° C.).

Extraction of Work by LP stage of the Main Turbine (a_6 - a_7)

Upon exiting the MSR **50**, the MSR exhaust steam **70** feeds into the LP stage **40** of the main turbine **10** graphically shown as a_6 on FIG. 9a. Work is extracted in the LP stage **40** that is graphically shown as going from a_6 to a_7 , which is then directed to the condenser **80**. The line representing the condenser **80** is also indicated on FIG. 9a.

GRAPHICAL REPRESENTATION OF PLANT PERFORMANCE IMPROVED BY THE INSTANT INVENTION

The Mollier chart for the typical steam cycle of a light water reactor electrical generation station with instant invention installed is illustrated in FIG. 9b. The diagram in FIG. 9b corresponds to equipment that is illustrated in FIG. 4.

Enhancement of Main Steam (b_1 - b_3)

Referring now to FIG. 9b, main steam **1** enters the inlet to the HPMS **105** at point b_1 as wet main steam **1** where it is superheated to point b_3 , and exits as superheated HP inlet mixed steam **130**.

Expansion in the HP stage of the Main Turbine (b_3 - b_4)

The HP inlet mixed steam **130** then enters the HP stage **35** of the main turbine **10** where it expands and work is extracted until it exits as HP exhaust steam **45**. As illustrated on FIG. 9b, this represents a decrease in quality from superheated condition to about 0.94. On FIG. 9b this is illustrated as going from point b_3 to point b_4 , and corresponds to going from a_1 to a_2 in a conventional plant as illustrated in FIG. 9a.

Removal of Moisture from HP exiting wet steam in the MSR (b_4 - b_5)

In the MSR **50** the moisture **55** from HP exhaust steam **45** is removed by chevron separating elements **57** that increase the quality to about 1.0 that is graphically represented in FIG. 9 by the line from point b_4 to b_5 .

Removal of Moisture from HP exiting wet steam in the MSR (b_5 - b_6)

After the moisture **55** is separated from the HP exhaust steam **45**, it is heated in two stages by high pressure mixer separator (HPMS) outlet steam **135**. This removes all remaining moisture and superheats the HP exhaust steam **45** to point b_6 . The HP exhaust steam **45** is typically at a quality of 0.94 at this point. At point b_5 the temperature is about 390° F. (200° C.). If the embodiment without an LPMS **140** is used, the HP exhaust steam **45** is typically at a temperature of about 550° to 600° F. (288°-316° C.) at this point. At point b_6 a temperature is about 550°-600° F. (288°-316° C.).

Mixing and Reheating of MSR exiting wet steam in the LPMS (b_6 - b_8)

After exiting the MSR **50** the MSR discharge steam **137** is mixed and reheated in the low pressure mixer superheater (LPMS) **140** by HRB low pressure steam **145** where it is fed into the LP stage **40** of the main turbine **10**.

Extraction of Work by LP stage of the Main Turbine (b_8 - b_9)

Work is being extracted in the LP stage **40** and is represented by line b_8 to b_9 on FIG. 9b. The line representing the condenser **80** is indicated on FIGS. 9a and 9b.

Elimination of the MSR

An additional advantage of the invention is elimination of the MSR **50**. When the conditioning of the steam in going from b_6 to b_8 is greater than the conditioning that occurs in going from b_4 to b_6 , the MSR **50** is no longer necessary. LP Stage utilizes Superheated Steam (b_8 to b_9).

Still another advantage of the invention is that the LPMS discharge steam **150** that is utilized by the LP stage **40** of the main turbine **10** is superheated for a longer portion of travel through the LP stage **40**, than that of the MSR exhaust steam **70**. This reduces the erosion of the LP stage **40** blades (not shown). Referring to FIG. 9a and FIG. 9b the distance from b_8 to b_8 is much larger than the distance from a_6 to a_6 , each distance being a diagrammatic representation of the path in which work is extracted from the superheated steam within the LP stage **40** of the main turbine **10**.

REDUCTION OF CORROSION TO STEAM GENERATOR

The corrosion of secondary components in any nuclear power plant is temperature dependent. This dependence is illustrated in FIG. 7. The most detrimental aspect of corrosion in a nuclear power plant steam generator 5 is tube failure due to corrosion. As illustrated in FIG. 7, with temperatures below 600° F. (315° C.), plugging versus years of operation only slightly increases. However, above 600° F. (315° C.) plugging increases dramatically.

Pressurized Water Reactors

Referring now to FIG. 8a, the instant invention enables a reduction in T_{hot} 701, which is due to the lowering of the temperature differential, $T_{hot}-T_{cold}$ (701-702). For a given thermal output of the reactor 700, assuming that (1) the mass flow rate 705 in the primary cycle is held constant, and (2) that T_{cold} 702 constant is held constant, as illustrated by the following equation:

$$Power_{reactor} = C_p * mass_{flow\ rate} * (T_{hot} - T_{cold}),$$

where C_p is the specific heat of the primary system coolant.

Boiling Water Reactors

Referring now to FIG. 8b, the instant invention enables a reduction in T_{hot} 801, which is due to the lowering of the temperature differential, $T_{hot}-T_{cold}$, (801-802) for a given thermal output of the reactor, assuming that (1) the mass flow rate in the primary system is held constant, and (2) that T_{cold} is held constant, as illustrated by the previous equation, except where C_p is the specific heat of the system coolant. The reactor 800 is a boiling water reactor, and T_{hot} is considered to be the average temperature at the outlet of the reactor 800 and the inlet to the steam generator 5 which in a boiling water reactor is one unit. The thermal efficiency is higher; consequently for a lower value of T_{hot} the same power level of the rated unit may be maintained. The reactor thermal power level can be reduced, the fuel cycle length can be extended, as well as a reduction in reactor vessel embrittlement.

FUEL CYCLE AND ENRICHMENT EFFECTS

Still another advantage of the instant invention is to decrease the enrichment requirements of the reduced thermal power of nuclear reactors. The conversion from thermal energy to electrical energy is typically from 33-35% for plants without the instant invention and about 42-45% for plants with the invention.

Since the nuclear reactor thermal energy output is directly keyed to how much nuclear fuel is present in the reactor and at what fission rate it is operated at, less fissions will be required. Any given fuel assembly has so many fissions possible over the life of the plant therefore the life of the plant is extended.

The reduction in the requirement of thermal power allows for the reduction of fuel enrichment.

Efficiency Without the Instant Invention

Net efficiency for a nuclear plant without the invention can be expressed by the following equation:

$$Efficiency = v_{net} = \frac{\text{Plant Electrical Output}}{\text{Plant Thermal Output}} = \frac{MW_{E-gen}}{MW_{Rx}}$$

where MW_{E-gen} is the total electrical output of the plant, and MW_{Rx} is the total heat output of the reactor.

Efficiency With the Instant Invention

In a nuclear plant utilizing the instant invention, the net efficiency can be expressed by the following equation:

$$v_{net} = \frac{\text{Plant Electrical Output}}{\text{Total Thermal Output}} = \frac{MW_{E-gen} + MW_{E-tur}}{MW_{Rx} + MW_{tur-fuel}}$$

where MW_{E-tur} is the output that the electrical generator of gas turbine generator 155 unit supplies to the grid 25, and $MW_{tur-fuel}$ is the energy release by the fuel in the burner 170 of the gas turbine 125. The increase in v_{net} is due to both the powers added by the gas turbine generator and the lowering of the MW_{Rx} for a given MW_{E-gen} .

Since the nuclear reactor thermal output is directly keyed to how much nuclear fuel is present in the reactor and at what fission rate it is operated at, less fissions will be required. Any given fuel assembly has so many fissions possible over the life of the plant; therefore, the life of the plant is extended while maintaining the same amount of electrical power 157 supplied to the grid 25.

Simulated computer runs on cost and efficiency have indicated that a 41.5% efficiency is possible when the instant invention is used on a 1260 MW PWR plant with 3570 MW_{th}, which normally has an efficiency of 34%. In this computer simulation, T_{hot} went from 615° F. (323° C.) to a T_{hot} of 597° F. (314° C.).

OPERATIONAL REQUIREMENTS

There are many types of nuclear electrical generation plants now in operation. Each plant will have its individual operational requirements that can influence how the instant invention is applied to that particular system. The main factors that would influence the selection of the most applicable embodiment include but are not limited to: (1) steam pressure requirements; (2) steam temperature requirements; (3) steam quality requirements; (4) type of heat recover boiler (HRB) being used (i.e., does it have a plurality of steam discharge pressures); (5) maintenance condition of the equipment, for example does the moisture separator reheater (MSR) need replacing; and (6) cost and availability of the equipment needed to install that particular embodiment.

Referring now to FIG. 13, an embodiment is illustrated that utilizes a heat recovery boiler that has only one steam pressure 107. This embodiment would use a second high pressure line 111 of the heat recovery boiler high pressure steam line 107. The second high pressure steam line 111 will require a pressure reduction device 112. In this embodiment the high pressure stage exhaust steam 45 is mixed with a second heat recovery boiler high pressure steam to form the low pressure mixer superheater (LPMS) discharge steam 150.

Referring now to FIG. 14, an embodiment is illustrated that utilizes a heat recovery boiler that has two steam pressures; i.e., the HRB high pressure steam 107 and the HRB low pressure steam 145. This embodiment would mix

the HP exhaust steam 45 directly with HRB low pressure steam 145 to form the low pressure mixer superheater discharge steam.

SELECTION OF STEAM MIXING EQUIPMENT

Some of the individual components utilized in the instant invention contains the same type of equipment and materials found in many existing combined cycle plants, as well as some found in existing nuclear power plants. The physical construction of these components is not material to the instant invention.

There are many options for the selection of the mixer superheaters that are utilized in the instant invention in addition to the MSR's 50 discussed above. Specifically, the high pressure mixer superheater 105 and the low pressure mixer superheater 140.

FIG. 10 illustrates a bellows 1010 connection for a tube 1020 injection of 107 through a nozzle 1030 into main steam 1 to form HP inlet mixed steam 130. The bellows arrangement 1010, or some other mechanism to allow for expansion and contraction, is necessary for all steam systems. Bellows arrangements 1010 typically have an upper hatch 1040 to allow for servicing. Typically an expanded portion of piping 1050 is installed to facilitate the insertion of the tube 1020

FIG. 11 illustrates a honeycomb mixer placed in an expanded portion of pipe down stream of where 107 enters the main steam 1 flow. This type of mixer-superheater typically resembles a piping tee, with pipe 1110 being the vertical leg and the expanded section 1120, between the inlet 1130 and the outlet 1140 forming the horizontal leg.

FIG. 12 illustrates a vortex-vane mixer in which 107 a main steam 1 is fed into a series of vanes 1210 that induces a clockwise pattern, with the pipe and vane arrangement of FIG. 12, that causes 107 and 1 to mix. The resulting mixture, HP inlet mixed steam 130, is typically removed by pipe in the plane perpendicular to the plane of HRB high pressure steam 107 and main steam 1.

These same type of mixers can be utilized in the mixer assembly 1507 that is used in the micro-jet mixer as the HPMS 105 that is discussed below.

MICRO-JET MIXER

Now referring to FIGS. 15a and 15b, the HPMS 105 is a micro-jet mixer that is particularly suited to achieving thorough mixing with a minimal pressure drop. In the first stage of the mixing process the micro-jet mixer mixes main steam 1 with HRB outlet steam 107 to form a combined steam flow 1501. The micro-jet mixer is a cylindrical tube and shell arrangement in which steam flows 1,107 are directed together in a mixing area 1503 wherein a combined steam flow 1501 is formed.

In the second stage the combined steam flow 1501 flows through baffle slots 1529 into a timing chamber 1505 which allows further mixing, thereby forming mixed steam flow 1533.

The final stage of mixing is done by directing the mixed steam flow 1533 through a mixer assembly 1507 mounted on or in an exit guide 1509, and then finally exiting the exit chamber 1511, where it becomes HP inlet mixed steam 130, which is finally directed via main steam piping 1513 to the steam turbine 10.

The micro-jet mixer and its major components are preferably cylindrical in shape. However, other shapes are within the contemplation of the practice of the invention. Particu-

larly those that are substantially circular or have a regular geometric shape.

Micro-Jet Mixer Detailed Flow Path

Main steam 1 enters the mixer shell 1515 at the inlet chamber 1516 via the main steam inlet 1517 and then flows around the slotted baffle 1527 into the mixing area 1503. The HRB outlet steam 107 enters first through the outer annulus 1519 via a HRB steam inlet 1521 and then through micro-jets 1523 into the mixing area 1503. The typical arrangement of HRB steam inlets 1521 is to have a plurality of inlets distributed axially around the circumference of the mixer shell 1515. FIG. 15a illustrates two HRB steam inlets 1521 diametrically disposed one to the other.

The combined flow 1501 proceeds in the space between the perforated baffle 1525 and the slotted baffle 1527 to the baffle slots 1529, which is referred to as the mixing area 1503. After the combined flow 1501 passes through the micro-jets 1523, it passes into the timing chamber 1505 which is defined as the space between the perforated baffle 1525 and the slotted baffle 1527.

The slotted baffle 1527 has both an attaching end 1528 and an interior end 1530. The interior end 1530 is typically plugged to force the mixed flow 1533 into the mixer assembly 1507. The attaching end 1528 is typically attached to the main steam outlet 1537. The baffle slots 1529 are typically cut in the attaching end 1528 of the slotted baffle 1527. The direction of the combined flow 1501 in the timing chamber 1505 is substantially counter to the direction of the mixed flow 1533 in mixing area 1503. Once the combined flow 1501 has gone through the baffle slots 1529 it is then termed to be mixed flow 1533.

Located within the slotted baffle 1527 is a substantially cylindrical exit guide 1509 of smaller overall diameter than the slotted baffle 1527. The mixed flow 1533 is then directed to a mixer assembly 1507 which is located at or in the interior end 1535 of the exit guide 1509. The mixer assembly 1507 can be simply the end of the exit guide 1509 or alternatively be vanes mounted inside the exit guide 1509. However, the mixer assembly will typically be of the type illustrated in FIGS. 10, 11, or 12.

The mixed flow 1533 is then directed to a mixer assembly 1507 which can be either mounted on or in the interior end 1535 of the exit guide 1509. After passing through the mixer assembly 1507 the mixed steam 1533 is referred to as HP inlet mixed steam 130.

The HP inlet mixed steam 130 then exits the exit chamber 1511 into main steam piping 1513 at the main steam outlet 1537 where it is directed to the steam turbine 10. It is within the contemplation of the instant invention that a plurality of exit chambers 1511 can be utilized. Specifically, now referring to FIG. 18a, outlet steam 135 is extracted from the micro-jet mixer and is used for conditioning within the MSR 50.

The embodiments illustrated in FIGS. 15a and 15b are with a single perforated baffle 1525 and a single slotted baffle 1527, however, multiple baffles 1525, or multiple baffles 1527, or any combination can be used as needed to increase the degree of mixing.

Thermal Differentials

Due to the temperature differentials expected, the perforated baffle 1525 should be mounted to the main steam inlet 1517 by means of a thermal expansion joint 1539. Thermal

expansion joints are well known in the art, and will allow for expansion and contraction without damage of the components of the micro-jet mixer.

Slotted Baffle Construction

Now referring to FIG. 16 the slotted baffle 1527 will typically be a substantially cylindrical pipe into which is cut a plurality of baffle slots 1529. For use in a nuclear power steam plant the baffle slots 1529 should be approximately 5 cm wide and 20 cm long, and substantially uniformly distributed transversely around the exit end 1543 of the slotted baffle 1527. The baffle slots 1529 can also be ellipsoid in shape with the major axis being 12 cm and the minor axis being 6 cm in length. The density of the baffle slots 1529 is determined by the design limitations on pressure drop. The more baffle slots 1529 would cause a reduction in pressure differential across the slotted baffle 1527.

Perforated Baffle Construction

Referring now to FIGS. 17a and 17b the perforated baffle 1525 is a substantially cylindrical into which is cut a plurality of micro-jets 1523. The micro-jets 1523 should be from 10 to 20 mm in diameter and substantially uniformly distributed over the entire surface of the perforated baffle. Or alternatively, the micro-jets 1523 can only be placed adjacent to the mixing area 1503. The density of the micro-jets 1523 is determined by the design limitations on pressure drop. The more micro-jets 1523 would cause a reduction in pressure differential across the perforated baffle 1525.

The micro-jets 1523 are orifices cut through the perforated baffle 1525 from the outer surface 1545 to the inner surface 1547. Additional control of the HRB outlet steam 107 velocity and pressure can be obtained by shaping the orifices as nozzles 1549. The nozzle configuration would be used in application where a lower pressure drop is required. Nozzles would typically have a larger inlet 1551 surface area than a outlet 1553 surface area. Other configurations for the orifices is also possible.

Examples of other possible orifice shapes include, rounded edges 1555, orifice walls 1557 can be concave, orifice walls 1557 can be convex, or where the orifice area on the outer surface 1545 is smaller than the orifice area on the inner surface 1547.

A possible added advantage of using particularly shaped orifices is that steam erosion of the surface would be less in more rounded walls 1557 and edges 1555.

Expected Equipment Sizes

Referring now to FIG. 15c, based upon a typical nuclear plant main steam line size of 0.8 meters the micro-jet mixer described above is expected to be from 2.5 to 3 meters in length (l_m , 1571) and about 1.6 meters in diameter (d_m , 1573). This size estimate is also based upon the use of four HRB high pressure steam 107 steam lines feeding the mixer 105.

A mixer 105 of this size will have a perforated baffle 1525 of about 1.3 meters and a slotted baffle 1527 of about 1.1 meters in diameter. This implies a distance (d_{sb-pb} , 1575) between the slotted baffle 1527 and the perforated baffle 1525 of about 100 mm before any allowance is made for the thickness of the materials of construction. The exit guide 1509, is expected to be about 0.8 meters in diameter (d_{eg} , 1585), causing a distance (d_{eg-ab} , 1577) between the exit guide and the slotted baffle of about 150 mm.

ELIMINATION OF THE LPMS

Referring now to FIG. 18a, still another embodiment of the instant invention is the elimination of the requirement of a LPMS 140. In this embodiment the MSR discharge steam 137 is directed immediately into the LP stage 40. This is possible because the HPMS outlet steam 135 has high enough enthalpy before entering the MSR 50 such that the MSR discharge steam 137 entering the LP stage 40 is of sufficient quality and enthalpy to achieve the corrosion and erosion benefits discussed above, while having sufficient enthalpy to prevent the desired work in the LP Stage 40. This increase in enthalpy is accomplished by means of a booster turbine 1801 in conjunction with the HRB 110 to treat, i.e. increase the enthalpy, of the HPMS outlet steam 135 prior to entering the HPMS 105. This is done by increasing the enthalpy of the HRB outlet steam 107. The enthalpy of the HRB outlet steam 107, which is at a high pressure, is increased by directing the HRB low pressure steam 145 to the booster inlet 1803 of the booster turbine 1801. The booster discharge steam (BDS) 1805 of the booster turbine 1801 is then directed to the HPMS 105 via the HRB 110. The pressure of the BDS 1805 is typically only higher than that of main steam 1 to facilitate mixing as discussed above. After leaving the HPMS 105 the 135 is directed to the MSR 50 thence directly to the LP stage 40.

The decrease in enthalpy of HRB low pressure steam 145 while in the booster turbine 1801 represents useful work and is used to produce electricity via generator 155. The shaft 1809 of the booster turbine will typically be mechanically connected to shaft of the gas turbine 125, but two shaft operation is also contemplated.

It should be noted that BDS 1805 is typically directly fed into the HPMS 105 within the HRB 110 where it is heated in a HRB tube assembly (not shown).

When the instant invention is operated in this mode it is the equivalent of using the booster turbine 1801 as a "high pressure stage", the HP stage 35 as a "medium stage", with the role of the LP stage 40 being unchanged.

It is within the contemplation of the practice of the instant invention that a multiplicity of booster turbines can be used to achieve higher enthalpy entering the micro-jet mixer used as an HPMS 105 or LP stage 40 as desired.

HPMS OR MIXER-JET VIBRATIONAL CONSIDERATIONS

It is well known in the art that large mass flow rates of steam mixing at different temperatures and pressures can cause equipment vibration. In most embodiments of the instant invention vibrational characteristics of the system will have to be considered.

When an HPMS 105 is used, the sizing of the components of the HPMS 105 can reduce the vibrations that can be experienced. Referring again to FIG. 15c, a mixing length l_m 1571 to micro-jet mixer diameter d_m 1573 ratio of approximately 15 to 1 will have advantageous vibrational characteristics.

Similarly, the slotted baffle length l_{ab} 1579 to slotted baffle diameter d_{ab} 1581 can be adjusted to cause minimal vibration. Also, the exit guide length l_{eg} 1583 to exit guide diameter d_{eg} 1585 can be adjusted to achieve minimal vibration.

Any time the length/diameter ratio is adjusted, due consideration should be given to the resulting volume. Volume not only influences the pressure drop and change, due to

15

friction or bernouli's effect respectively, but it is also important in allowing the desired mixing to take place.

For example, the volume V_{oa} of the outer annulus **1509** which is determined by the micro-jet mixer diameter d_m **1573**, the micro-jet mixer length l_m **1571**, and diameter of the perforated baffle d_{pb} **1587**, and for a circular micro-jet mixer can be expressed as

$$V_{oa} = l_m * \pi * 1/4 * (d_m^2 - d_{pb}^2),$$

is an important consideration. As is the volume of the timing chamber **1505**, exit chamber **1511**, guide chamber **1512**, and the inlet chamber **1516**. Each of these volumes can be expressed using same method of calculation and is well known in the art.

USE OF THE LPMS WITH THE BOOSTER TURBINE

To provide maximum plant flexibility it may be desirable to be able to have both a LPMS **140** and a booster turbine **1801** installed. Referring now to FIG. **18b** the LPMS **140** and the booster turbine **1801** can be used in conjunction with one another, or used alone, depending upon operational needs. In this embodiment the HRB high pressure steam **107** is directed to the booster turbine **1803** inlet where work is extracted as previously discussed.

In this embodiment steam will be extracted from the booster turbine **1801** in two ways. After work has been extracted the DBS **1805** will also be directed to the HRB **110**. However, at mid-turbine **1813**, extraction booster steam (EBS) **1815** will be extracted and directed to the HPMS **105**.

The HRB low pressure steam **145** is directed to the LPMS **140** in order to increase the enthalpy of the LPMS discharge steam **150** to the desired level.

The EBS **1815** extracted from **1803** will be at a slightly higher pressure than main steam **1** and is directed to HPMS **105**. From this point the flow is substantially the same as discussed above.

PIPING, EQUIPMENT, AND VALVE PERMUTATIONS

It is obvious to one skilled in the art that various combinations of the piping and equipment disclosed can be used. For example, all components and piping necessary for any embodiment can be installed. The use of valves will enable any specific embodiment to be achieved as desired.

OTHER EMBODIMENTS POSSIBLE

While several preferred embodiments of the present invention have been shown and described, it will be apparent to those skilled in the art that many changes and modifications may be made without departing from the invention in its broader aspects. The appended claims are therefore intended to cover all such changes and modifications as fall within the true spirit and scope of the invention.

We claim:

1. A method for improving the operational characteristics of a nuclear power electric generation plant that comprises the steps of:

- (a) inserting into a downstream portion of main steam piping from a steam generator in said nuclear power plant a micro-jet mixer;

16

- (b) supplying heat recovery boiler high pressure steam to said micro-jet mixer,

- (i) said heat recovery boiler high pressure steam being supplied from a heat recovery boiler, wherein said heat recovery boiler high pressure steam is mixed with said main steam causing said main steam to be superheated, thereby forming a high pressure inlet mixed steam;
- (ii) said heat recovery boiler being heated by a chemically fueled means;

- (c) directing said high pressure inlet mixed steam into a high pressure stage of a main turbine of said nuclear power electric plant, wherein work is extracted by said high pressure stage of said main turbine;

- (d) directing a high pressure exhaust steam into a moisture separator reheater wherein said high pressure exhaust steam is mixed with a micro-jet exhaust steam wherein a moisture separator reheater discharge steam is formed that is further superheated;

- (e) directing said moisture separator reheater discharge steam into a low pressure stage of the main turbine of said nuclear power electric generation plant;

whereby efficiency of said nuclear power electric generation plant is increased.

2. An apparatus for improving the operational characteristics of a nuclear power electric generation plant comprising:

- (a) a micro-jet mixer receiving main steam from a steam generator in said nuclear power plant;

- (b) a heat recovery boiler providing high pressure steam to said micro-jet mixer wherein said high pressure steam is mixed with said main steam and forms a high pressure inlet mixed steam that is superheated;

- (c) a high pressure stage of a main turbine of said nuclear power electric plant, wherein work is extracted from the high pressure inlet mixed steam leaving a high pressure exhaust steam;

- (d) a moisture separator reheater wherein said high pressure exhaust steam is mixed with a micro-jet mixer exhaust steam forming a moisture separator discharge steam;

- (e) a low pressure mixer superheater for mixing said moisture separator discharge steam with a heat recovery boiler low pressure steam forming low pressure mixer superheater discharge steam; and

- (f) a low pressure stage of the main turbine of said nuclear power electric generation plant for receiving said low pressure mixer superheater discharge steam and extracting work therefrom.

3. The apparatus as recited in claim 2 wherein the micro-jet mixer comprises:

- (a) a mixer shell having a cross section and having an inlet chamber for receiving main steam, an outlet chamber, and at least one HRB steam inlet for receiving heat recovery boiler outlet steam,

- (b) a perforated baffle located within said mixer shell having a plurality of micro-jets cuts therein, said micro-jets extending from an outer surface to an inner surface, wherein said heat recovery boiler outlet steam flowing along said outer surface, and said main steam flowing along said inner surface, wherein when said heat recovery boiler outlet steam exits said micro-jets and enters said main steam a combined flow is formed;

- (c) a slotted baffle located within said perforated baffle having an interior end and an attaching end,

17

- (i) said attaching end being attached to a main steam outlet, and having a plurality of baffle slots cut therein for receiving said combined flow,
- (ii) said combined flow flowing through said baffle slots, thereby forming a mixed flow;
- (d) an exit chamber located within said slotted baffle having a mixer end, a discharge end, and a chamber region,
- said mixed flow directed to said mixer end thereby forming a high pressure mixed steam, which is directed to a main steam piping;
- wherein said high pressure mixed steam is directed to the high pressure inlet of said main turbine.
4. The apparatus in claim 3 wherein the cross-section is substantially circular in shape.
5. The apparatus in claim 3 wherein the cross-section is substantially in the shape of a regular polygon.
6. The apparatus in claim 3 having a plurality of slotted baffles.
7. The apparatus in claim 3 having a plurality of perforated baffles.
8. The apparatus in claim 3 wherein a plurality of exit chambers are used.
9. The apparatus in claim 3 wherein the mixer end of the exit chamber comprises a mixer assembly.
10. The apparatus in claim 8 wherein a mixer assembly is constructed of honeycomb mixing material.
11. The apparatus in claim 8 wherein the mixer assembly is a vortex mixer.
12. The apparatus in claim 3 wherein the micro-jets have a diameter from 10–20 mm.
13. The apparatus in claim 3 wherein the micro-jets are nozzle shaped with a nozzle inlet and a nozzle outlet.
14. The apparatus in claim 13 wherein the nozzle inlet is larger than the nozzle outlet.
15. The apparatus in claim 3 wherein the baffle-slots are rectangular slots of 5 cm in width and 10 cm in length.
16. The apparatus in claim 3 wherein the baffle-slots are ellipsoid in shape with the major axis being 12 cm and the minor axis being 6 cm in length.
17. A method for improving the operational characteristics of a nuclear power electric generation plant that comprises the steps of:
- (a) inserting into a downstream portion of main steam piping from a steam generator in said nuclear power plant a high pressure mixer superheater;
- (b) supplying heat recovery boiler high pressure steam to said high pressure mixer superheater,
- (i) said heat recovery boiler high pressure steam being supplied from a heat recovery boiler, wherein said heat recovery boiler steam is mixed with said main steam causing said main steam to be superheated;
- (ii) said heat recovery boiler being partially heated by exhaust from a chemically fueled turbine;
- (iii) said heat recovery boiler being partially heated by a booster turbine discharge steam;
- (c) directing said superheated main steam into a high pressure stage of a main turbine of said nuclear power electric plant, wherein work is extracted by said high pressure stage of said main turbine;
- (d) directing a high pressure exhaust steam into a moisture separator reheater then entering a low pressure mixer where the separated high pressure exhaust steam is mixed with steam from a low pressure heat recovery boiler; and
- (e) directing said moisture separator reheater steam to a low pressure stage of the main turbine, wherein further

18

- work is extracted by said low pressure stage of said main turbine and efficiency of said nuclear power electric generation plant is increased.
18. A method for improving the operational characteristics of a nuclear power electric generation plant that comprises the steps of:
- (a) inserting into a downstream portion of main steam piping from a steam generator in said nuclear power plant, a high pressure mixer superheater,
- (i) said high pressure mixer superheater mixing said main steam with a extraction booster turbine discharge forming a high pressure mixture superheater outlet steam,
- (ii) said extraction booster turbine discharge being supplied by a booster turbine,
- (iii) said extraction booster turbine discharge steam and said main steam forming a high pressure exhaust steam;
- (b) directing said high pressure exhaust steam and said high pressure mixture superheater outlet steam into a moisture separator reheater thereby forming a moisture separator discharge steam;
- (c) combining said moisture separator discharge steam with a low pressure heat recovery boiler discharge steam, in a low pressure mixer superheater forming a low pressure mixer superheater discharge steam;
- (d) said low pressure heat recovery boiler discharge steam being formed by,
- (i) supplying heat recovery boiler high pressure steam, from a heat recovery boiler, to a booster turbine,
- (ii) directing the booster discharge steam back to said heat recovery boiler,
- (iii) treating said booster discharge steam in said heat recovery boiler supplying heat recovery boiler high pressure steam to said high pressure mixer superheater,
- (iv) said heat recovery boiler being partially heated by exhaust from a chemically fueled turbine,
- (v) said heat recovery boiler being partially heated by a booster turbine discharge steam;
- whereby efficiency of said nuclear power electric generation plant is increased.
19. A method for improving the operational characteristics of a nuclear power electric generation plant that comprises the steps of:
- (a) inserting into a downstream portion of main steam piping from a steam generator, in said nuclear power plant, a micro-jet mixer;
- (b) supplying heat recovery boiler high pressure steam to said micro-jet mixer,
- (i) said heat recovery boiler high pressure steam being supplied from a heat recovery boiler, wherein said heat recovery boiler steam is mixed with said main steam causing said main steam to be superheated, thereby forming a high pressure inlet mixed steam;
- (ii) said heat recovery boiler being heated by a chemically fueled means;
- (c) directing said high pressure inlet mixed steam into a high pressure stage of a main turbine of said nuclear power electric plant, wherein work is extracted by said a high pressure stage of said main turbine;
- (d) directing a high pressure exhaust steam into a moisture separator reheater where said high pressure exhaust steam is mixed with a micro-jet mixer exhaust steam wherein a moisture separator discharge steam is formed;

19

(e) directing said moisture discharge steam to a low pressure mixer superheater where it is mixed with heat recovery boiler low pressure steam forming low pressure mixer superheater discharge steam;

(f) directing said low pressure mixer superheater discharge steam into a low pressure stage of the main turbine of said nuclear power electric generation plant; whereby efficiency of said nuclear power electric generation plant is increased.

20. The method in claim 1 wherein said chemically fueled means is a turbine that drives a generator that supplies electricity.

21. The method in claim 20 wherein said turbine is a gas driven turbine.

20

22. The method in claim 20 wherein said turbine is a fossil fueled turbine.

23. The method in claim 1 wherein a high pressure stage exhaust steam is mixed with a second heat recovery boiler high pressure steam to form the low pressure mixer superheater discharge steam.

24. The method in claim 1 wherein a high pressure stage exhaust steam is mixed with heat recovery boiler low pressure steam to form the low pressure mixer superheater discharge steam.

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